

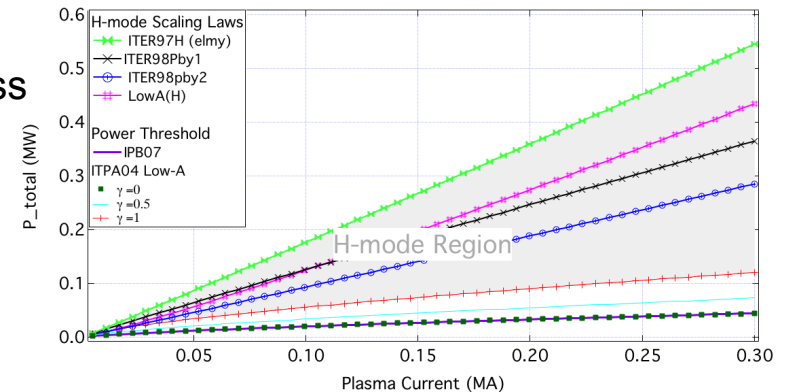


# Experiment Overview



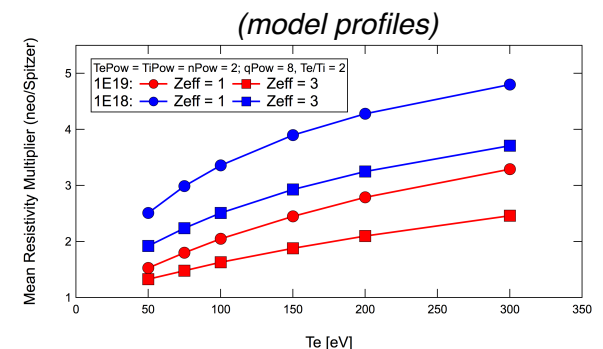
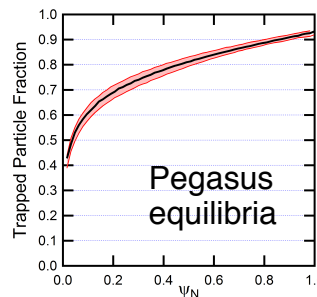
# Operation at $A \sim 1$ Offers Ready Access to Advanced Tokamak Physics

- Very low  $B_t$  at modest  $I_p$  = very low  $P_{th}$  for H-mode access
  - High edge shear = separatrix not necessarily needed
  - Easy access with ohmic heating only
  - **BUT** need hot edge => centerstack fueling



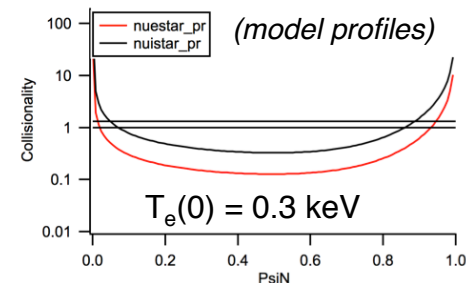
H-mode power threshold and ohmic confinement scalings for PEGASUS.

- Short connection lengths and very strong trapping = neoclassical effects at low  $T_e$ 
  - High particle trapping fractions
  - Strongly non-Spitzer resistivity
  - Bootstrap current possible



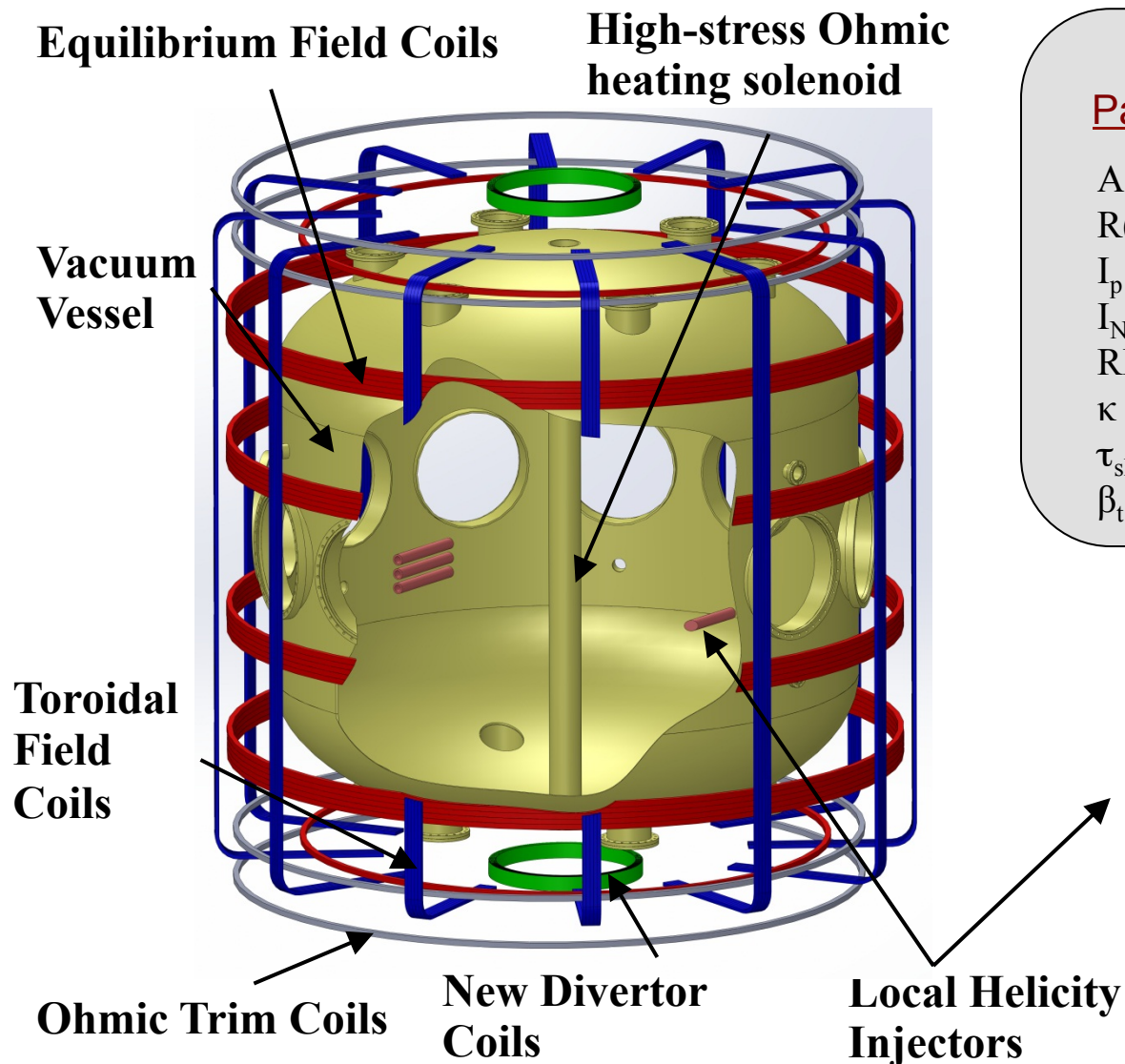
- High  $j_{edge}/B_t$  plus H-mode pedestal = Peeling mode and peeling-ballooning modes accessible

- With short pulse and low  $\langle T_e \rangle$ , easy diagnostic accessibility
  - e.g., probes in pedestal region



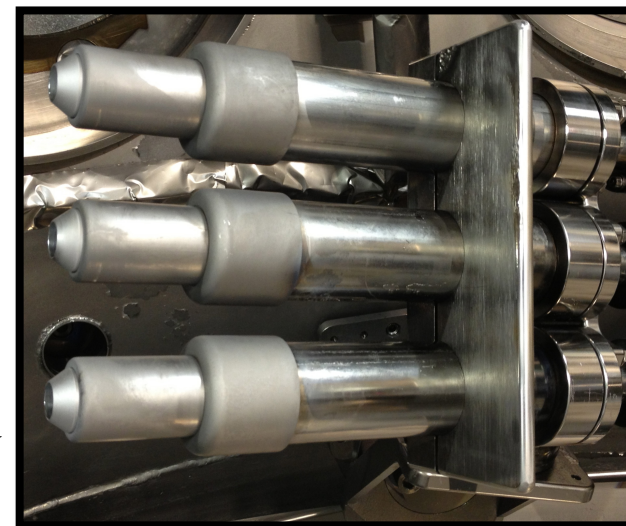


# Pegasus is a Compact, Ultralow-A ST



## Experimental Parameters

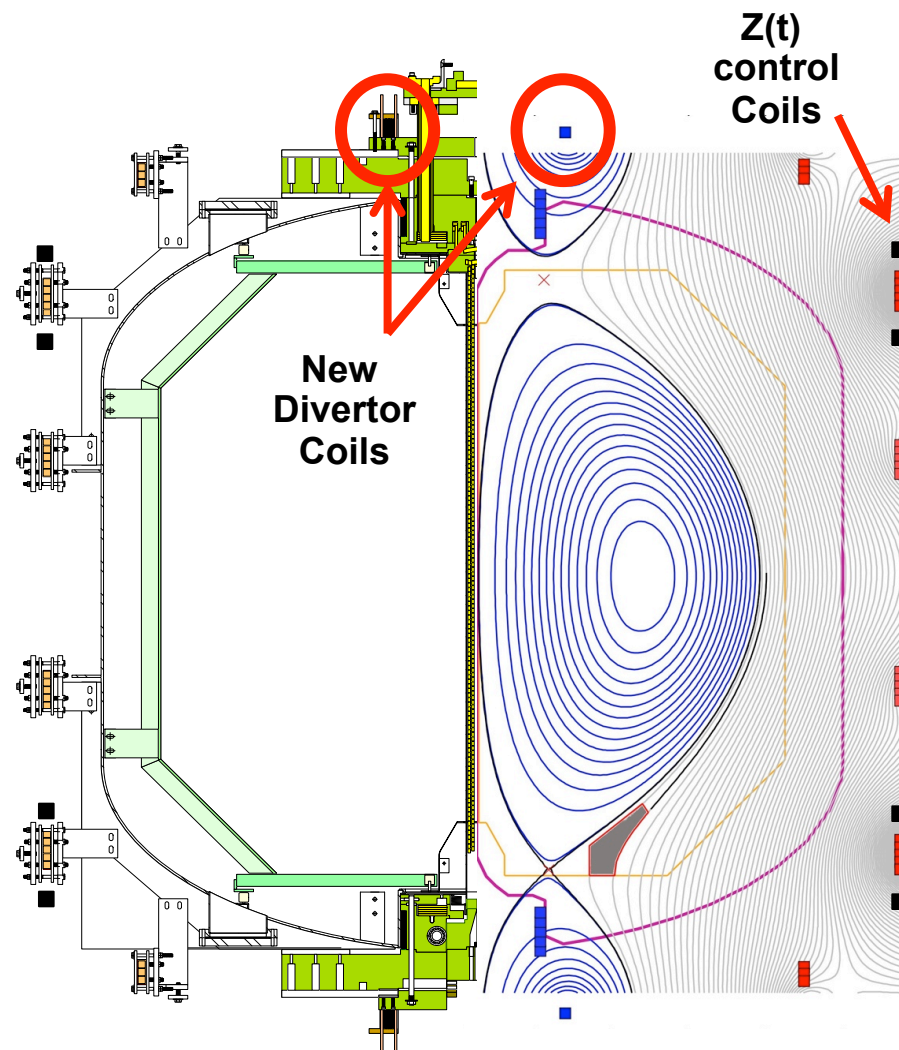
<u>Parameter</u>	<u>Achieved</u>	<u>Goals</u>
A	1.15 – 1.3	1.12 – 1.3
R(m)	0.2 – 0.45	0.2 – 0.45
$I_p$ (MA)	$\leq .23$	$\leq 0.30$
$I_N$ (MA/m-T)	6 – 14	6 – 20
$RB_t$ (T-m)	$\leq 0.06$	$\leq 0.1$
$\kappa$	1.4 – 3.7	1.4 – 3.7
$\tau_{\text{shot}}$ (s)	$\leq 0.025$	$\leq 0.05$
$\beta_t$ (%)	$\leq 25$	$> 40$





# Device Upgrades Support Expanded Helicity Injection, Edge Physics Studies

- Helicity Injection Systems
  - Injector material, design optimization: *reduced PMI*
  - Active fueling control
  - Multi-aperture injector array for high- $I_p$  startup
- Power Supplies, Heating, Fueling
  - New helicity injection power:  $2.2\text{ kV}$ ,  $14\text{ kA}$  supply
  - Centerstack fueling: *LHI fueling and H-mode access*
- Expanded PF Coil Set and Control
  - New PF coils, power systems: *vertical control*
- Diagnostic Deployment and Improvements
  - Multipoint Thomson Scattering
  - High-speed  $T_i(R,t)$ : *Anomalous reconnection heating*
- New divertor coils → *separatrix operation*
  - Exploit H-mode operating regime
  - Flux expansion to optimize LHI startup

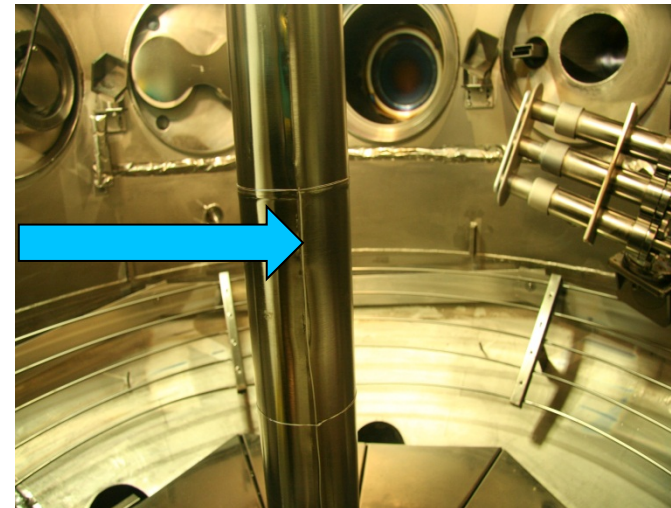
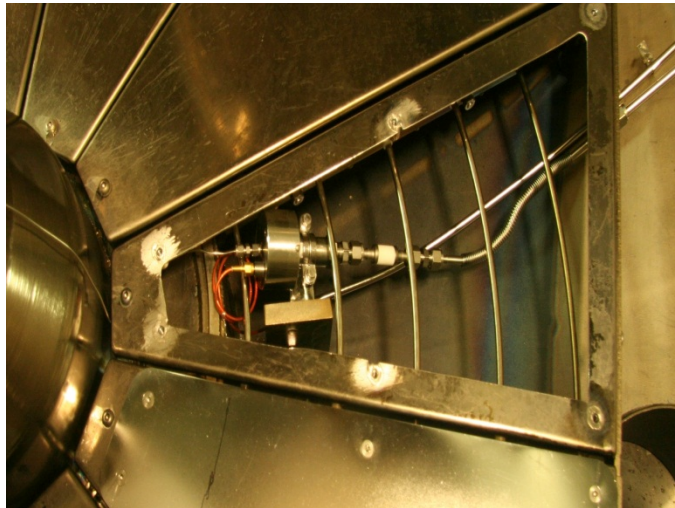
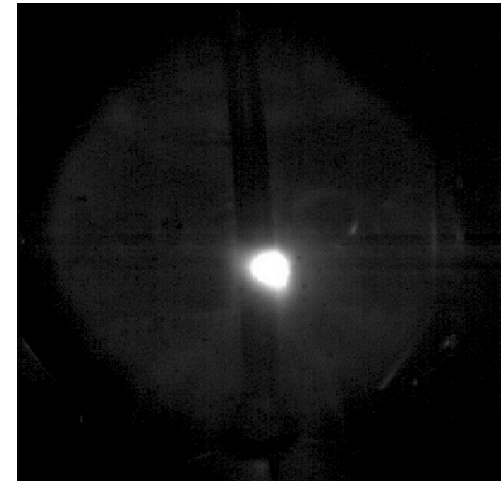






# Implemented Internal High-field Side Gas Valve

- Characteristics
  - In-vacuum piezoelectric valve
  - $z = -30$  cm
  - Throughput 300-3000 Torr\*L/s
  - Stabilized with heat shielding





$L \rightarrow H$  Power Threshold as  $A \rightarrow 1$



# Power threshold required to enter H-mode

- Sufficient power must be applied to trigger a transition from L-mode to H-mode
  - Power can be provided by NBI, ECH, ICH, LH, OH
  - Also achieved by biasing the plasma using an external electrode or by biasing a limiter
  - $P_{\text{thres}}$  depends strongly on  $n_e$ ,  $B_{\text{TF}}$ , and ion  $\nabla B$  drift direction
- Nature of transition is still under investigation due to lack of first-principles model with predictive capability
  - E.g. predator-prey, flow shear



# L-H Power threshold scaling from experiment data

- Recommended scaling from Progress on ITER Physics Basis (2007), high-A  $P_{\text{thres}}$  scaling

- This equation has 21.4% RMSE and no ST data

$$P_{PIP B} = 0.042 n_{20}^{0.73} B_{TF}^{0.74} S^{0.98}$$

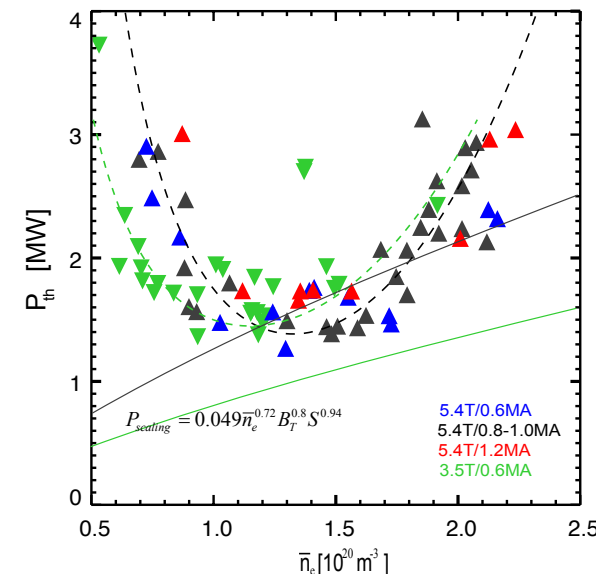
$$S = 4\pi^2 a R \left( \frac{1 + \kappa^2}{2} \right)^2$$

- Earlier high-A scaling (ITPA, 2004)

- MAST ( $A \approx 1.45$ ) requires 1.6 x more power,  
NSTX ( $A \approx 1.32$ ) requires 3.7 x more power

$$P_{04} = 0.06 n_{20}^{0.7} B_{TF}^{0.7} S^{0.9}$$

- More complicated:  $P_{\text{th}}$  has nonlinear dependence on  $N_e$



Y. Ma, et al, "Scaling of H-mode threshold power and L-H edge conditions with favourable ion grad-B drift in Alcator C-Mod," Nuclear Fusion, vol. 52, no. 2, p. 023010, 2012.





## Different empirical scaling explicitly incorporates aspect ratio

$$P_{thr\_low-A} = 0.072 n_{20}^{0.7} |B|_{out}^{0.7} S^{0.9} \left( \frac{Z_{eff}}{2} \right)^{0.7} F(A)^\gamma$$

$$|B|_{out} = (B_{tout}^2 + B_{pout}^2)^{0.5}, \quad B_{tout} = B_{TF} \frac{A}{A+1}, \quad B_{pout} = \frac{\mu_0 I_p}{2\pi a} (1 + \varepsilon),$$

$$F(A) = \frac{0.1A}{1 - \left( \frac{2}{1+A} \right)^{0.5}} \quad \text{and} \quad \gamma = 0.5 \pm 0.5$$

- But this equation essentially has a 100% error bar on A, demonstrating more low-A data could be valuable

I. H. mode Power Threshold Database and T. p. b. Takizuda, "Roles of aspect ratio, absolute B and effective Z of the H-mode power threshold in tokamaks of the ITPA database," Plasma Physics and Controlled Fusion, vol. 46, no. 5A, pp. A227–A233, 2004.

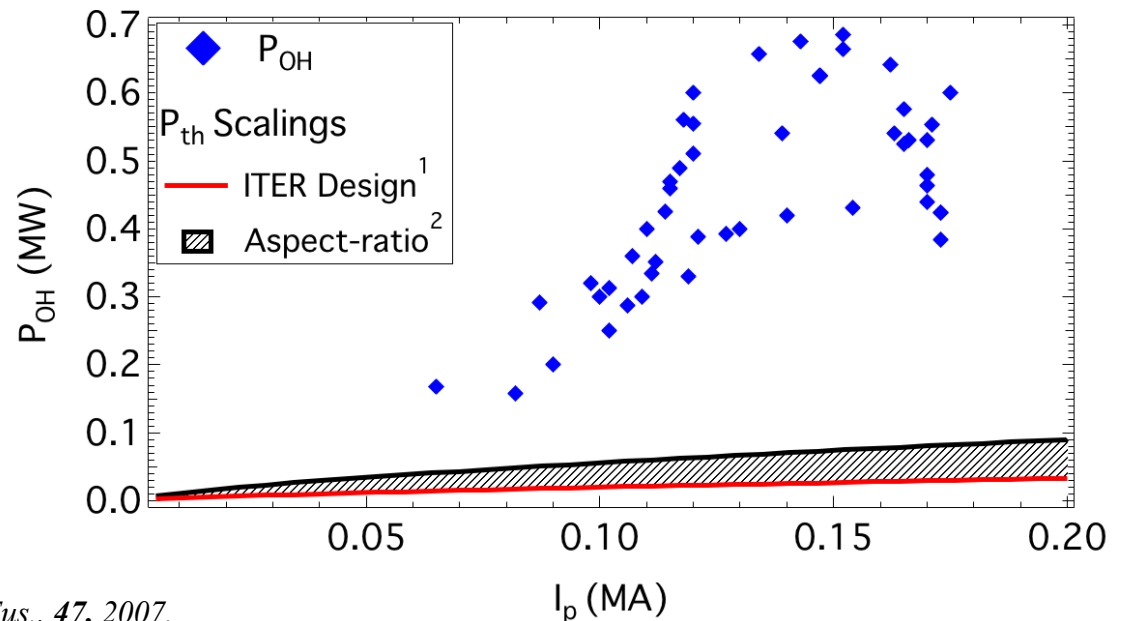


# PEGASUS $P_{OH}$ Exceeds $P_{th}$ Predictions

- L-H power threshold scalings:  $P_{th} \sim n_e^{0.7} B_T^{0.7} S$ 
  - At very low-A and hence low  $B_T$ ,  $P_{th}$  is very low
  - Scalings<sup>1,2</sup> suggest PEGASUS  $P_{th} < 0.1$  MW
  - $P_{OH} = 0.2\text{--}0.7$  MW
- Modest  $t_{shot}$  and  $\langle T_e \rangle$  allow probes in pedestal

## Experimental Parameters

Parameter	Achieved
$B_T$ (T)	0.08–0.16
A	1.15–1.3
R (m)	0.2–0.45
$I_p$ (MA)	$\leq 0.21$
$\kappa$	1.4–3.7
$t_{shot}$ (s)	$\leq 0.025$
$T_e$ (eV)	100–200



<sup>1</sup> Accepted ITER design threshold  $P_{th}$ : K. Ikeda, "Nucl. Fus.", **47**, 2007.

<sup>2</sup>  $P_{th}$  with low-A data: I. H. mode Power Threshold, Plasma Phys. Control. Fus., **46**, 2004

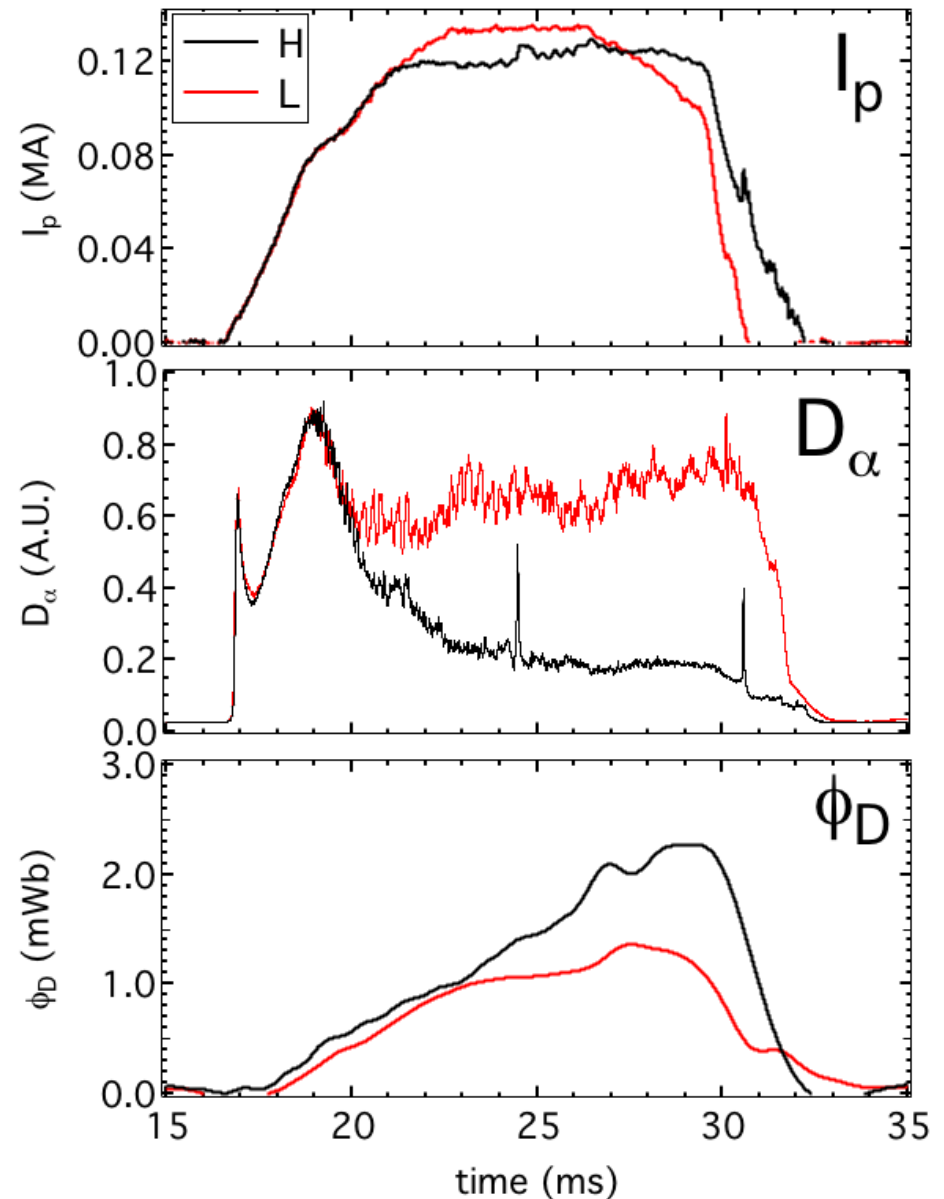
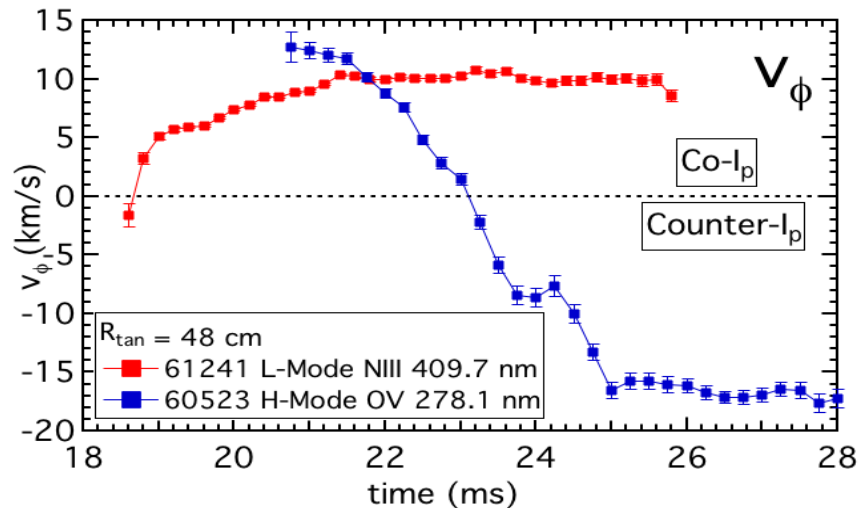


# H-mode in Ohmic Plasmas



# Ohmic H-mode Plasmas have Standard Signatures

- H-mode signatures observed:
  - Quiescent edge
  - Increased core  $T_e$ ,  $T_i$  inferred
  - Reduced  $D_\alpha$
  - Large and small ELMs suggested
  - Bifurcation in  $\phi_D$
  - Core  $v_\phi$  reverses





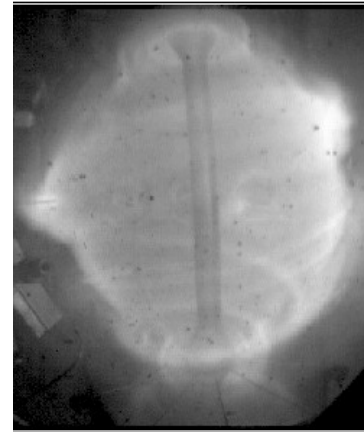


# Fueling Location, Particularly in STs, is Critical for Achieving H-mode

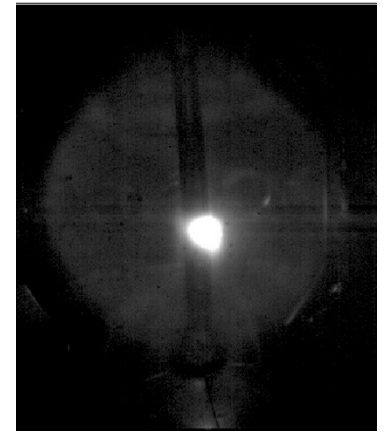
- LFS and HFS fueling
- H-mode achieved using HFS fueling
  - Similar to MAST and NSTX<sup>1</sup>
  - Both limited and diverted

## Limited

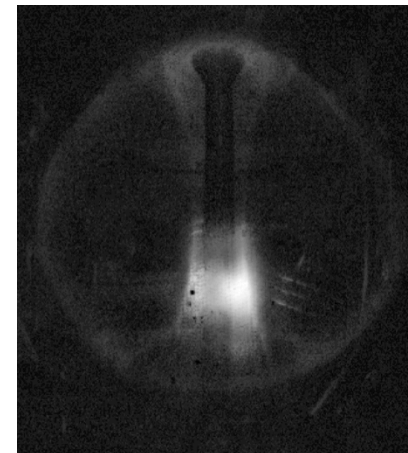
LFS Fueled (L)



HFS Fueled (H)



## Diverted (H)

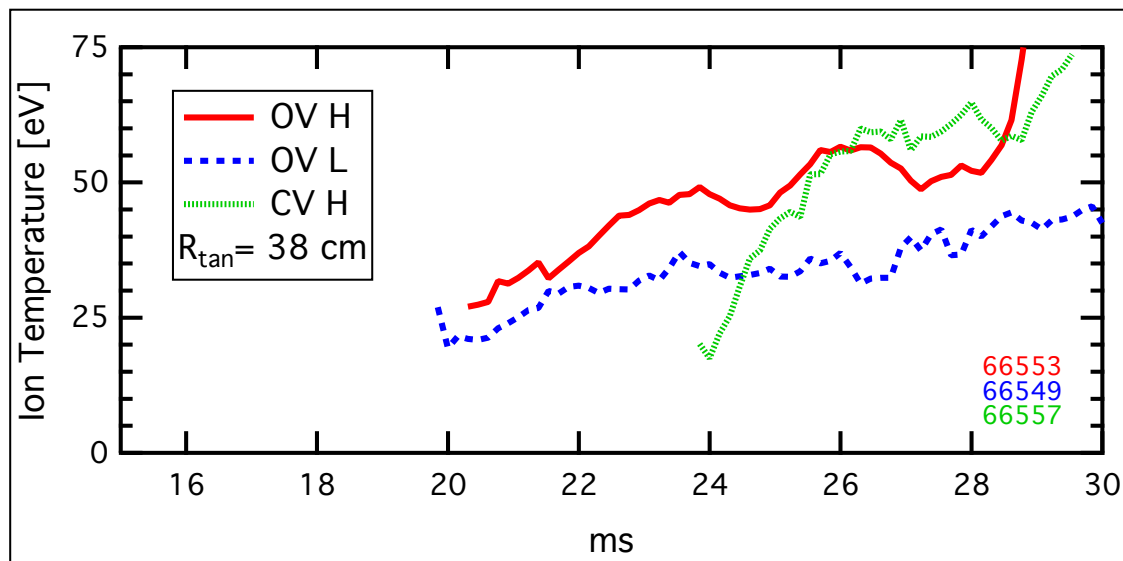


<sup>1</sup> A. R. Field et al, *Plasma Phys. Control. Fus.*, **46**, 2004.



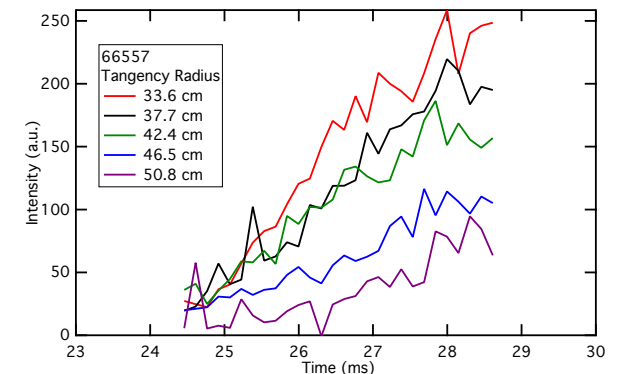
# Impurity Spectroscopy Suggests $T_i$ and $T_e$ Increase in H-mode Core Region

- Chord-integrated  $T_i(t)$  increases in H-phase
- Appearance of CV in H-phase only indicates increased  $T_e(0,t)$



- CV not present in L-mode discharges
  - CV I.P. = 392 eV
  - OV I.P. = 113 eV

- CV intensity is centrally peaked

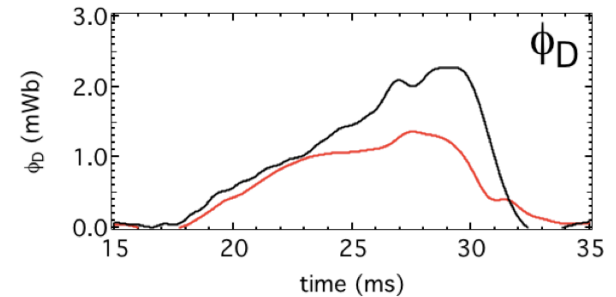




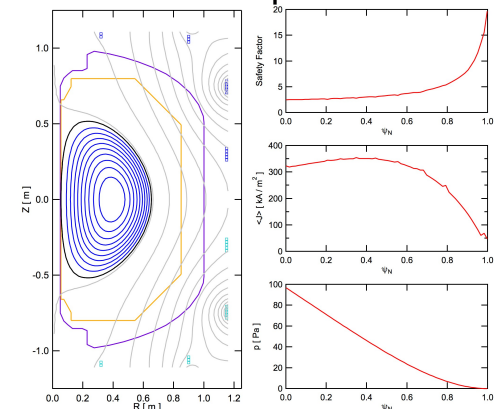
# Increased $I_i$ Indicated by Rise in Paramagnetism at L-H Transition

- Rise in diamagnetic flux loop signal indicates rise in *paramagnetism* at  $A \sim 1$ 
  - Not a rise in total stored energy
  - Magnetic reconstructions confirm increased  $I_i$  in H-phase
    - H:  $I_i \sim 0.45$ ; L:  $I_i \sim 0.35$
- At constant  $I_p$ ,  $V_{loop}$ , this suggests localized core plasma heating
  - Supported by indication of  $T_e(0)$  increase

H-mode indicated by rise in paramagnetism

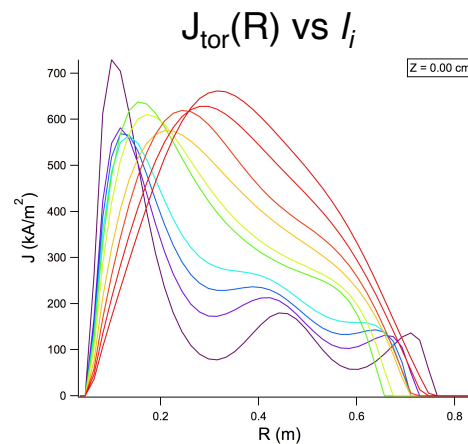
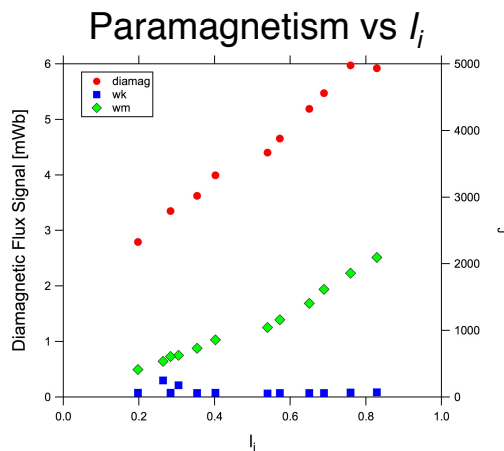


Model Equilibrium



Equilibrium Parameters  
Shot 12345, 0.000 ms

$I_p$	150 kA	$R_0$	0.354 m
$\beta_t$	0.0035	$a$	0.299 m
$I_i$	0.54	$A$	1.18
$\beta_p$	0.019	$\kappa$	1.7
$W$	1101 J	$\delta$	0.43
$B_{T0}$	0.177 T	$q_{95}$	11.6





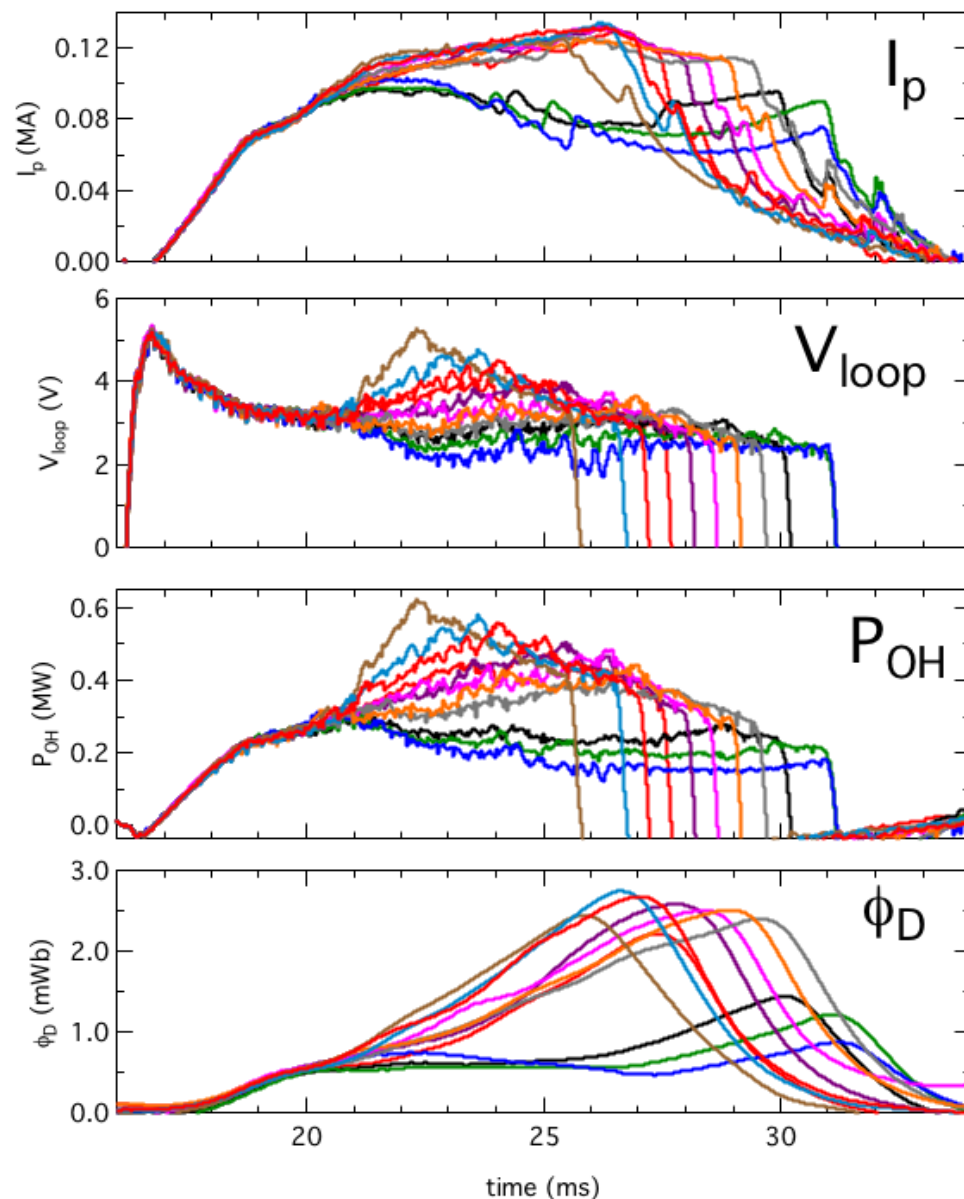
# $P_{th}$ and $J_{edge}(R)$ Pedestal





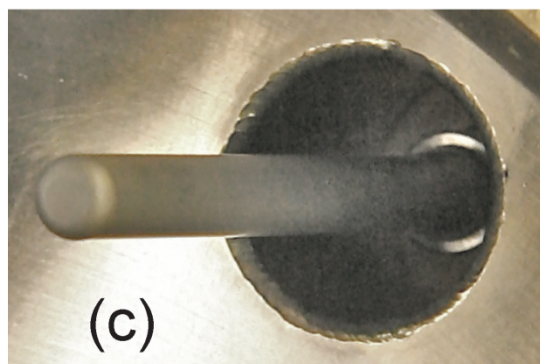
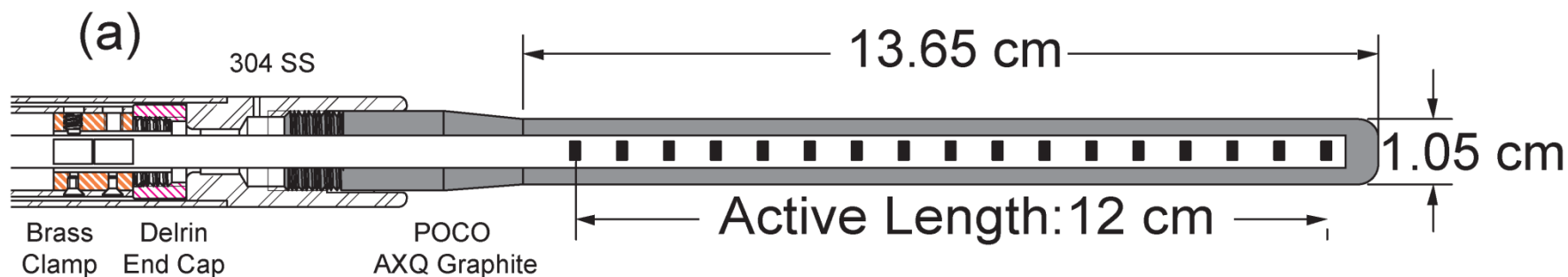
# $P_{th}$ Measured using $V_{loop}$ Scans

- Infer  $t_{LH}$  from bifurcation in  $\phi_D$ 
  - Vary  $P_{OH} = I_p V_{loop}$
  - Constant  $I_{EF}$ , shape, fueling
- $P_{th} \sim 0.25\text{--}0.30$  MW
  - Scalings predict  $< 0.1$  MW





# PEGASUS Hall Probe Deployed to Measure J



- Solid-state InSb Hall sensors
  - Sypris model SH-410
- Slim C armor as low-Z PFC
  - Minimizes plasma perturbation
- 16 channels, 7.5 mm radial resolution
- 25 kHz bandwidth



## $J_\phi(R,t)$ Calculable Directly from Ampère's Law

$$\mu_0 J_\phi = (\nabla \times \mathbf{B})_\phi = \frac{\partial B_R}{\partial Z} - \frac{\partial B_Z}{\partial R}$$

- Simplest test follows from  $B_R(Z)$  or  $B_Z(R)$  measurements
- Petty\* solves for an off-midplane  $B_Z(R)$  measurement set and an elliptical plasma cross-section:

$$\mu_0 J_\phi = -\frac{B_Z}{\kappa^2 (R - R_0)} \left( 1 - \frac{Z^2 R_0}{\kappa^2 R (R - R_0)^2} \right) - \frac{dB_Z}{dR} \left( 1 + \frac{Z^2}{\kappa^4 (R - R_0)^2} \right)$$

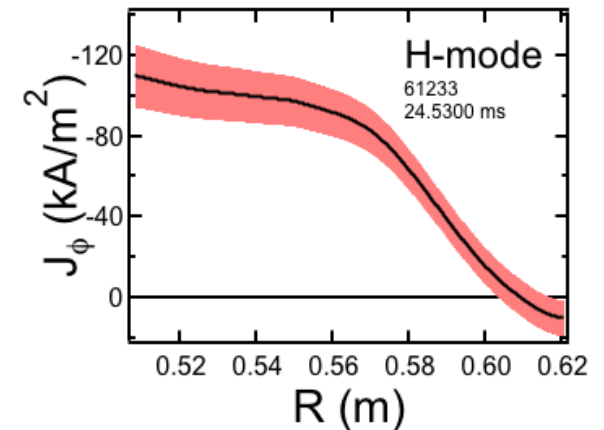
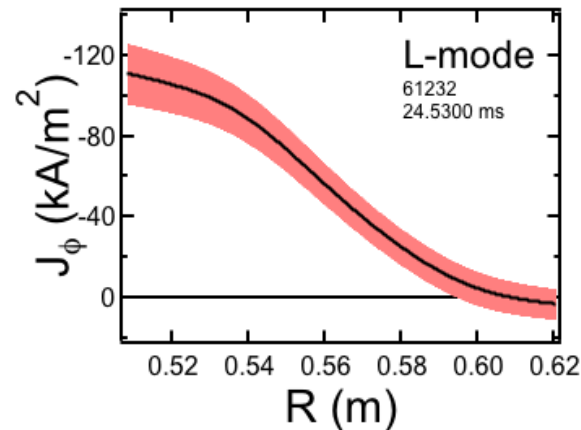
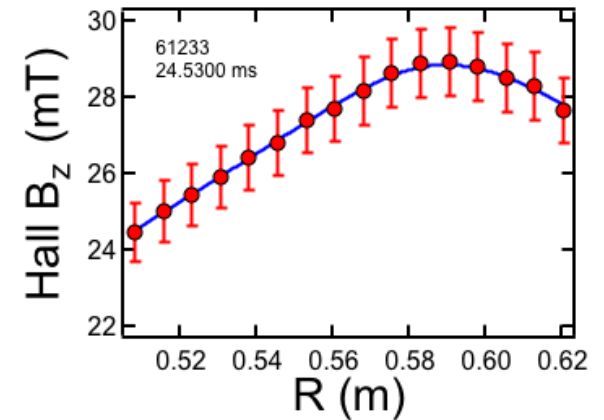
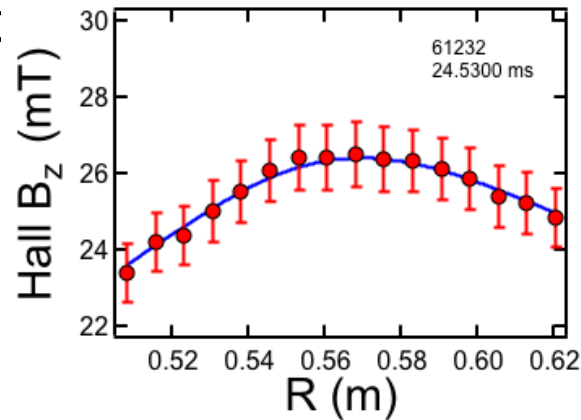
- Does not make assumptions on shape of  $J(R)$

\*: Petty, *et al.*, Nucl. Fusion **42**, 1124 (2002)



# Current Pedestal Measured using Hall Probe Array

- Internal  $B_z$  measurement from Hall probe array yields local  $J_\phi(R,t)$ <sup>1</sup>
- Current gradient scale length significantly reduced in H-mode
  - L  $\rightarrow$  H: 6  $\rightarrow$  2 cm
  - $\rho_i \sim 1.8$  cm



<sup>1</sup> M. Bongard, *Rev. Sci. Instrum.* **81**, 10E105 (2010).



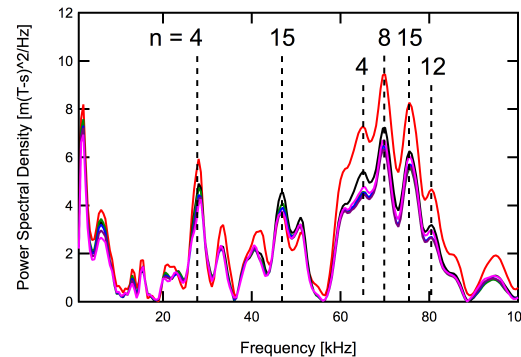


# General Observations

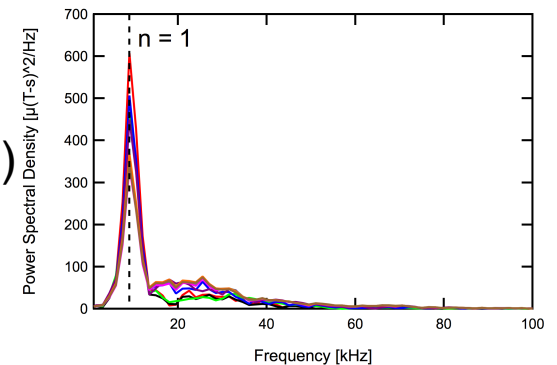


# Large and Small ELMs Suggestive of Type I and III ELMs are Seen

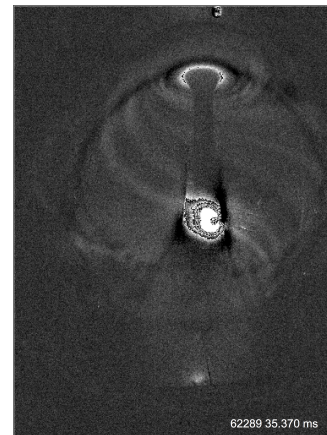
- Filament structures observed
  - Large ELMs infrequent and violent
    - Can cause H-L back-transition
    - Occur at high  $P_{OH}$
  - Small ELMs more ubiquitous and less perturbing
    - Occur at lower  $P_{OH}$
- $n$  measured with close-fitting coil array through ELM crash
  - PEGASUS results similar to NSTX
    - Large (“Type I”): intermediate- $n$
    - Small (“Type III”): low- $n$
  - STs appear to have structure opposite that of ATs



Large (Type I)



Small (Type III)

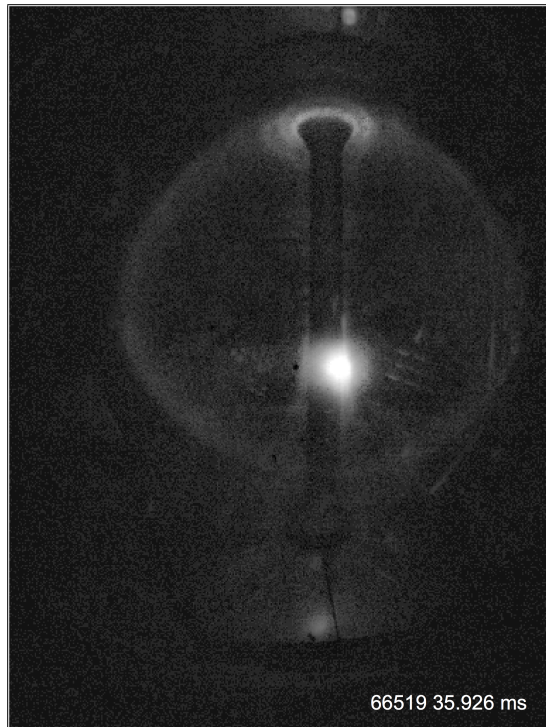


Coherent filaments associated with ELMs

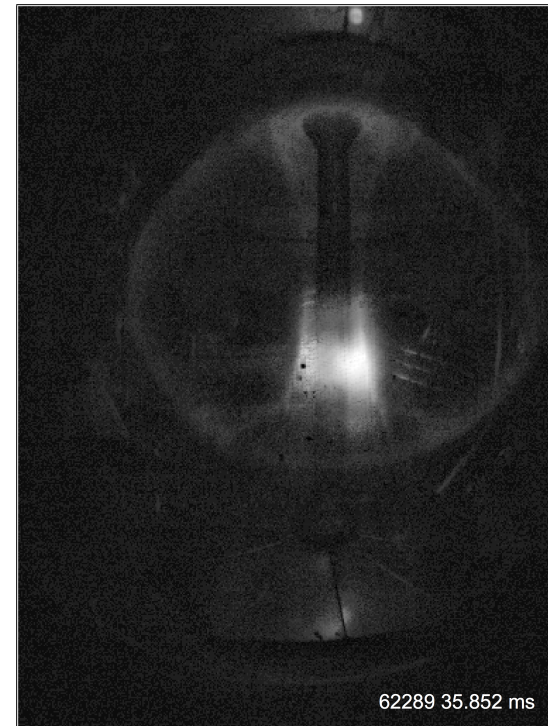


# Divertor Coils Activated to Access Standard Separatrix-Limited H-modes

Non-diverted:  
Centerstack Limited



Diverted:  
Separatrix Limited

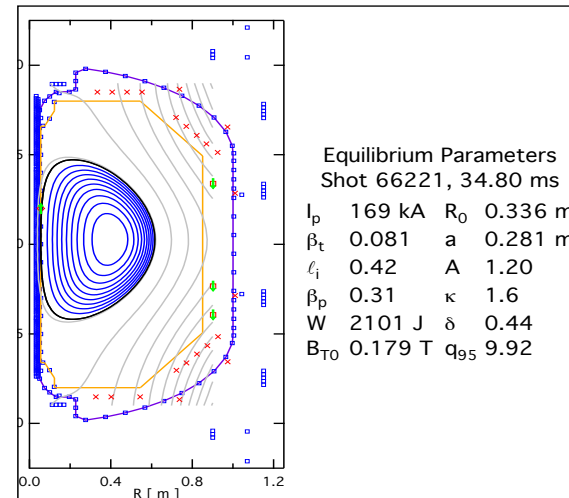
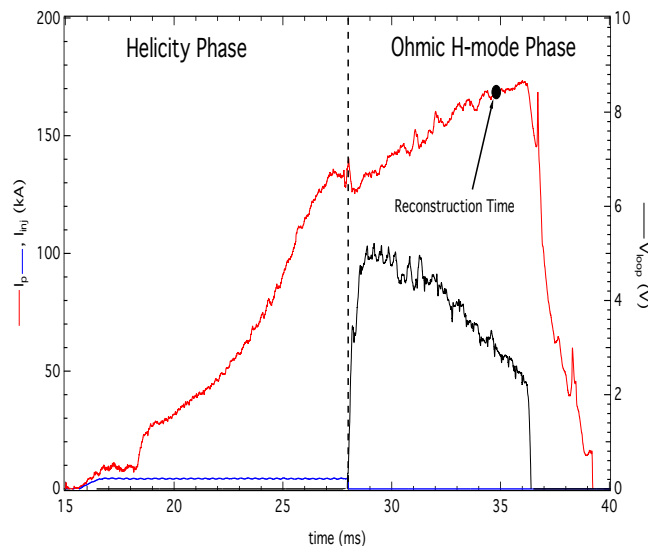


- Initial results: no clear difference between diverted and non-diverted
- *But*, short pulse length complicates  $\tau_E$  measurement



# LHI Startup Compatible with Consequent High-Quality OH H-mode

- High- $I_p$ , long-pulse H-mode plasmas desirable for Pegasus goals
  - Confinement and edge stability studies
  - Attaining high  $\beta_t$  regime
- Need additional current drive
  - LHI-initiated discharge readily couples to ohmically-driven H-mode

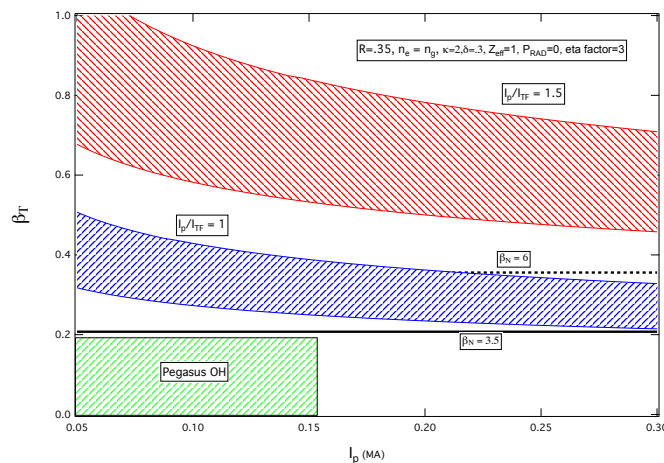


- But, difficult to raise  $I_p$  in ohmic phase with available V-sec
  - May be influenced by: residual MHD activity; increasing  $I_i(t)$
- High  $I_p$ , long-pulse operation awaits new integrated LHI assembly, power systems upgrades, and new OH solenoid

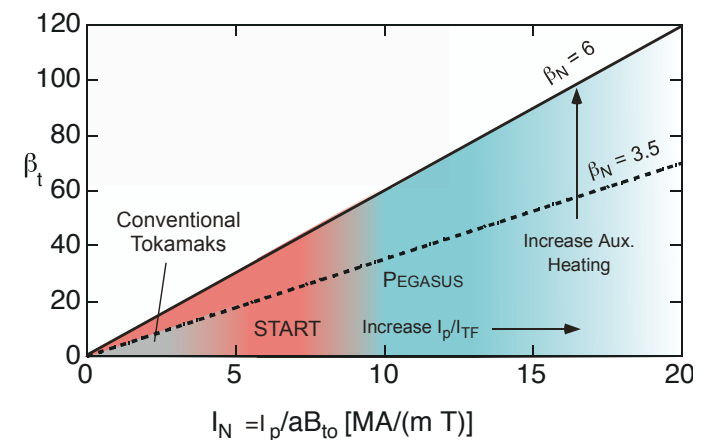
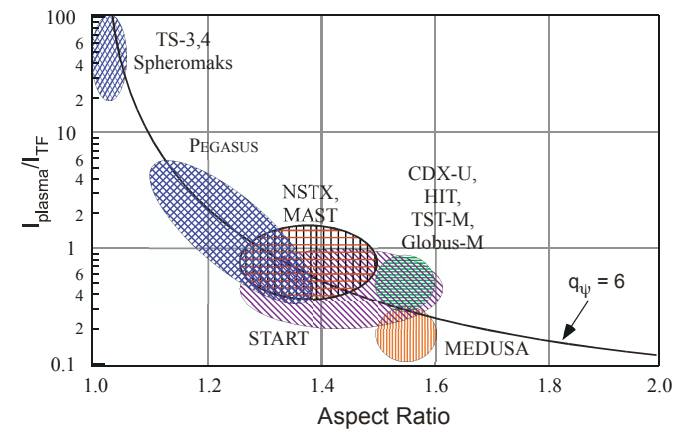


# Non-solenoidal startup and H-mode Facilitates Access to high- $\beta_t$ Regime as $A \rightarrow 1$

- Pegasus designed to explore tokamak stability limits at  $A \sim 1$
- Requires access to relevant space
  - High  $I_p/I_{TF}$
  - Good OH confinement



- Research thrusts should enable access to this unique stability regime
  - LHI: High  $I_p/I_{TF}$
  - H-mode: Good OH confinement





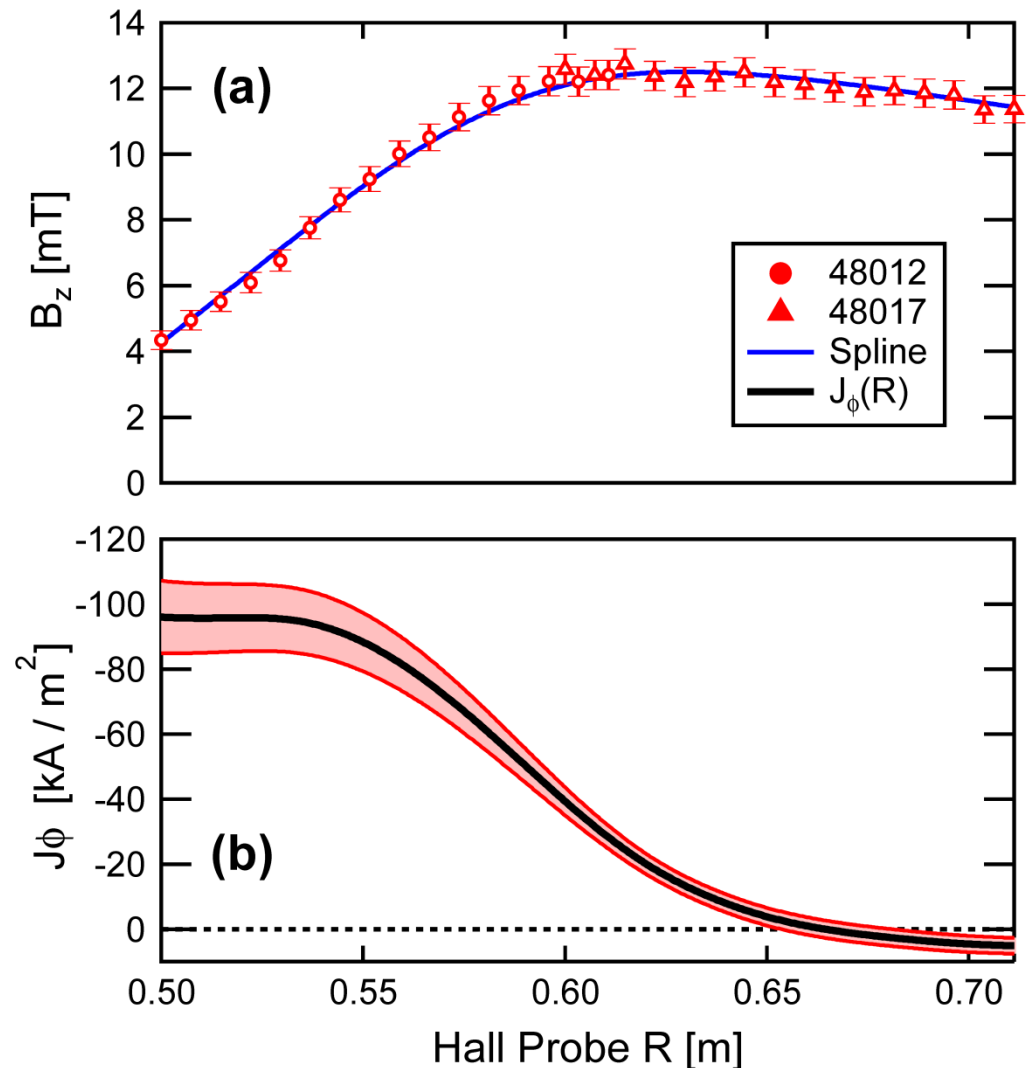
# Summary: A ~ 1 Operation Enables Studies of H-mode Phenomena

- Low toroidal field at A ~ 1 facilitates access to H-mode
  - $P_{th} \sim 5x$  greater than  $P_{th}$  scalings' predictions
  - Edge current pedestal observed
- Large, small ELMs observed and  $J_{edge}(R,t)$  dynamics measured
  - Clear difference in toroidal mode numbers between large and small ELMs
  - $J_{edge}(R, t)$  shows current-hole perturbation during ELMs
- Proposed upgrades will extend studies to wider parameter space
  - New OH solenoid: 5-6x V-sec increase (courtesy PPPL)
    - Increased pulse length, transport equilibrium; more relaxed  $J(R)$
  - 2x  $B_{tf}$  increase: vary  $P_{th}$ ; edge stability boundaries
  - Core and edge plasma diagnostics
    - Multipoint Thomson scattering
    - Edge electrostatic and magnetic probes
    - Core ion spectroscopy



# Direct $J_\phi(R)$ Profiles Obtained in PEGASUS

- Straightforward  $J$  estimation
  - Obtain Hall Probe  $B_z(R,t)$
  - Compute  $dB_z/dR$  using interpolated smoothing spline\*
  - Compute  $J_\phi(R,t)$  given geometry
- Resultant  $J_\phi(R,t)$  consistent with  $I_p$ , MHD evolution
- Radial span extendible with multi-shot averaging
- Higher-order shaping effects negligible within errors



Bongard *et al.*, Phys. Rev. Lett. **107**, 035003 (2011)

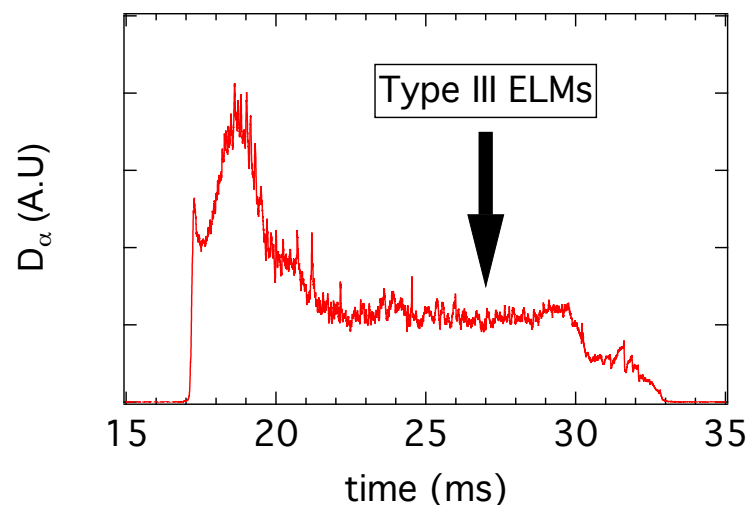
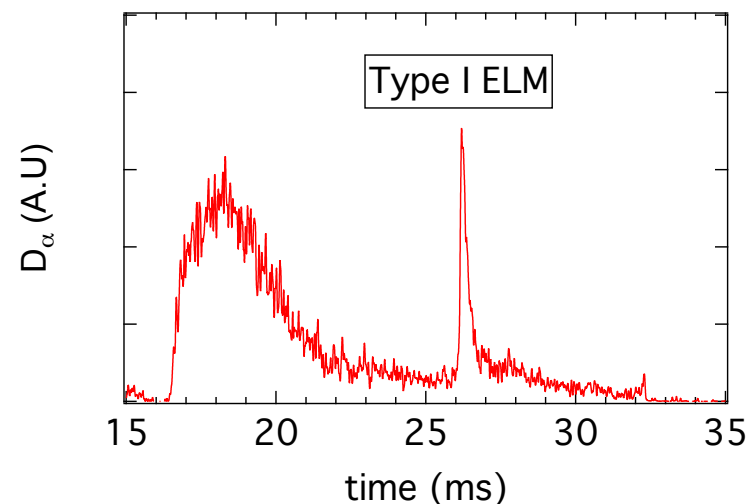
\*: Reinsch, Numerische Mathematik **10**, 177 (1967)





# Type I and Type III ELMs seen in H-mode

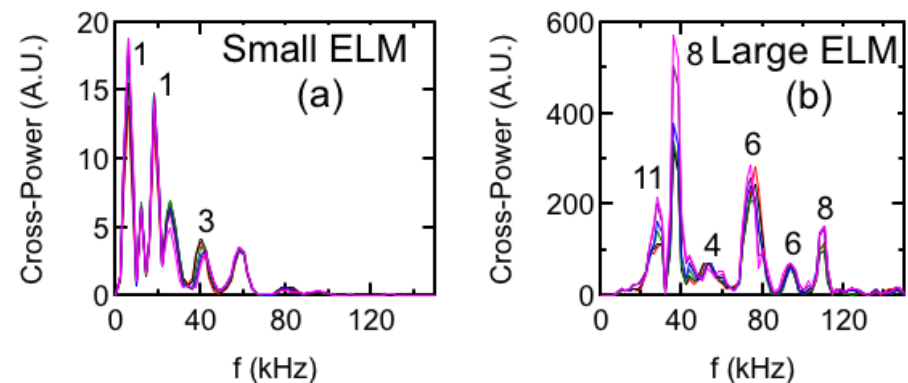
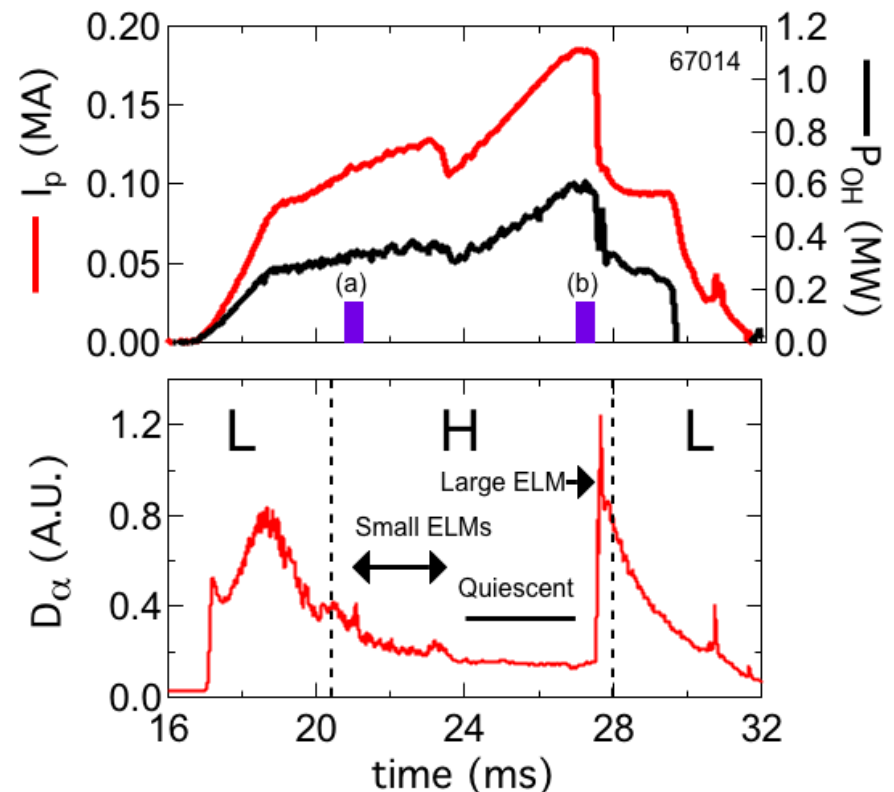
- Tentatively identified via magnetic signatures
  - Type I expected to have intermediate  $n$  modes
  - Type III expected to have low  $n$  modes
- Typically only one Type I ELM occurs in a discharge
- Many Type III ELMs occur in a single discharge





# Large and Small ELMs Suggestive of Type I and III ELMs are Seen

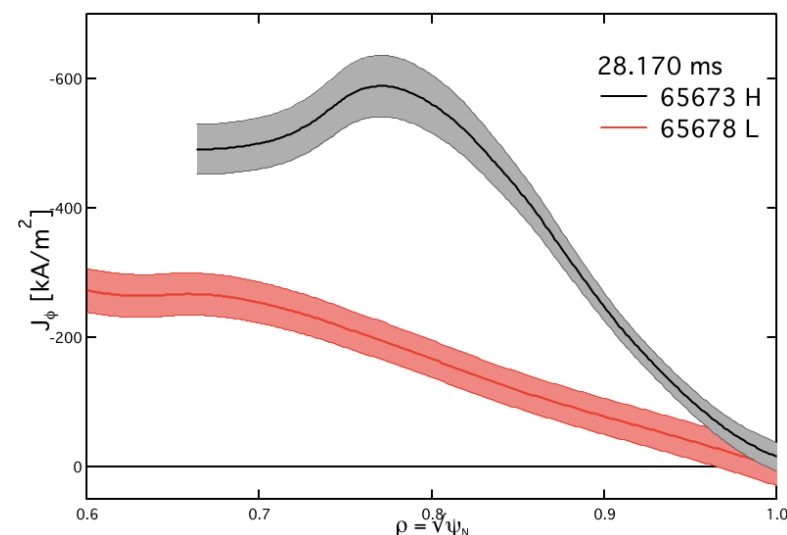
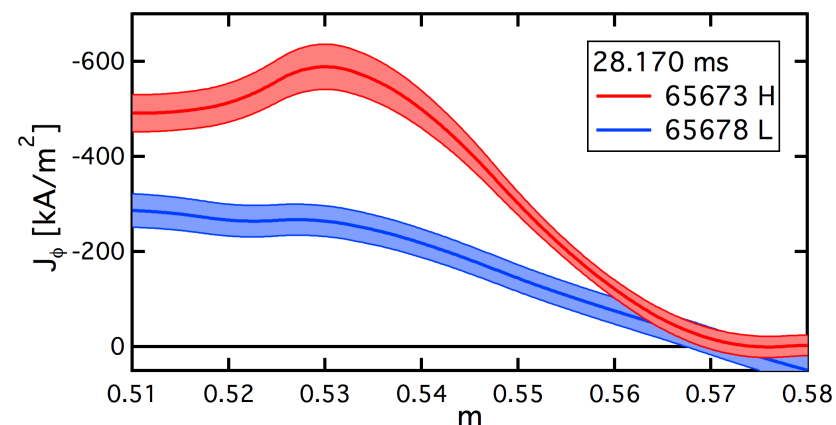
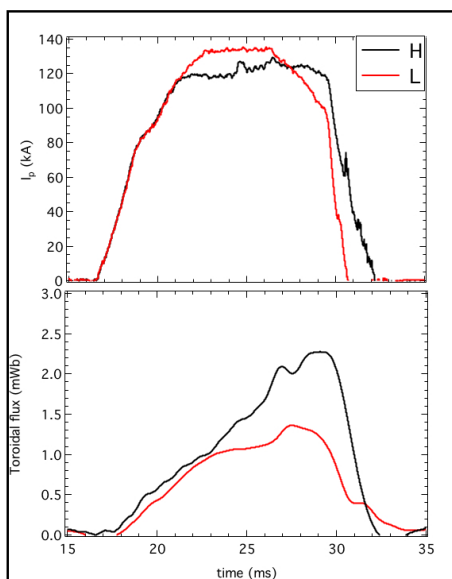
- Filament structures observed
  - Large ELMs infrequent and violent
    - Can cause H-L back-transition
    - Occur at high  $P_{OH}$
  - Small ELMs more ubiquitous and less perturbing
    - Occur at lower  $P_{OH}$
- $n$  measured with close-fitting coil array through ELM crash
  - PEGASUS results similar to NSTX
    - Large ("Type I"): intermediate- $n$
    - Small ("Type III"): low- $n$
  - STs appear to have structure opposite that of ATs





# Edge Current Pedestal Observed in H-Mode

- Internal B measurements from Hall array\* yield local  $J_\phi(R,t)^{**}$ 
  - Map to  $\psi_N$  only approximate
- Current gradient scale length significantly reduced in H-mode  
L  $\rightarrow$  H: 6  $\rightarrow$  2 cm



\*: M.W. Bongard *et al.*, Rev. Sci. Instrum. **81**, 10E105 (2010)  
 \*\*: C.C. Petty *et al.*, Nucl. Fusion **42**, 1124 (2002)