

# Non-Solenoidal Startup via Local Helicity Injection and Edge Stability Studies in the Pegasus Toroidal Experiment

Michael W. Bongard  
on behalf of the Pegasus Team



University of  
Wisconsin-Madison

Workshop on Exploratory  
Topics in Plasma and Fusion  
Research

Fort Worth, TX  
February 15, 2013



PEGASUS  
Toroidal Experiment



# Exploiting Unique Aspects of the ST to Improve Fusion Energy Science

- Non-solenoidal startup: Increasing reactor attractiveness
  - Local Helicity Injection produces tokamak plasmas using edge current drive
    - Predictive understanding through helicity conservation, Taylor relaxation constraints
  - Reduces cost, complexity of device
  - Technique applicable to any tokamak, not just ST
- Edge physics: Detailed measurements of pedestal, ELM dynamics
  - Low-A naturally provides access to peeling instability underlying ELMs
    - Simplified diagnostic access → unique  $J_{\text{edge}}(t)$  measurements
  - Extension to ITER-relevant peeling-ballooning physics via H-mode operation
- Testing boundaries of tokamak stability at ultimate geometric limit
  - High  $\beta_T$ , toroidal field utilization  $I_p/I_{TF}$  as  $A \rightarrow 1$





# Pegasus is a Compact, Ultralow-A ST

Equilibrium Field Coils

High-stress Ohmic heating solenoid

Vacuum Vessel

Toroidal Field Coils

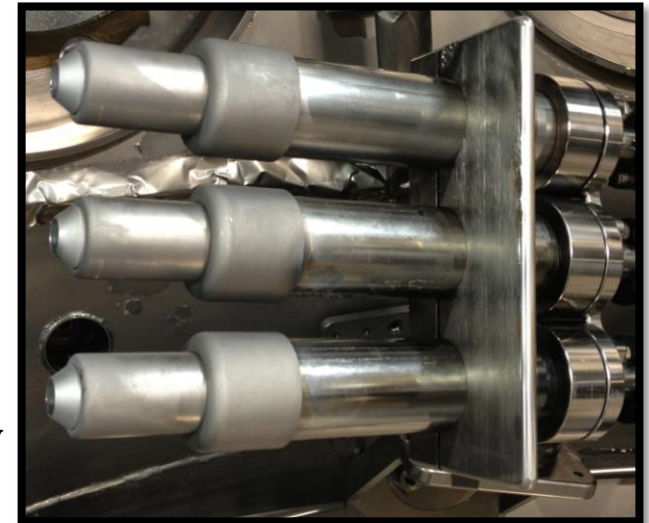
Ohmic Trim Coils

New Divertor Coils

Local Helicity Injectors

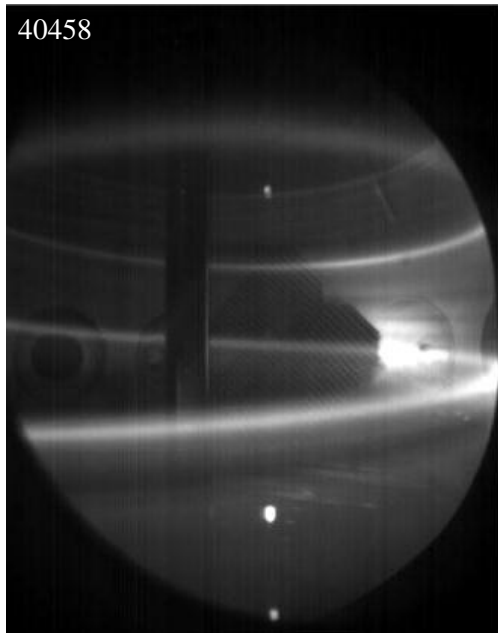
## Experimental Parameters

<u>Parameter</u>	<u>Achieved</u>	<u>Goals</u>
A	1.15 – 1.3	1.12 – 1.3
R(m)	0.2 – 0.45	0.2 – 0.45
$I_p$ (MA)	$\leq .23$	$\leq 0.30$
$I_N$ (MA/m-T)	6 – 14	6 – 20
$RB_t$ (T-m)	$\leq 0.06$	$\leq 0.1$
$\kappa$	1.4 – 3.7	1.4 – 3.7
$\tau_{\text{shot}}$ (s)	$\leq 0.025$	$\leq 0.05$
$\beta_t$ (%)	$\leq 25$	$> 40$





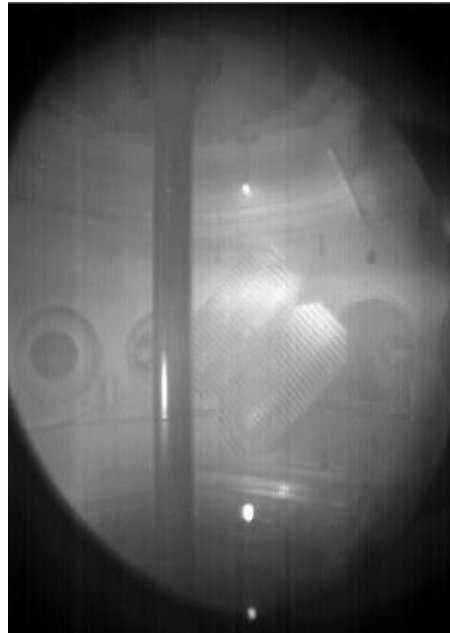
# Local Helicity Injection Offers Scalable Non-Solenoidal Startup



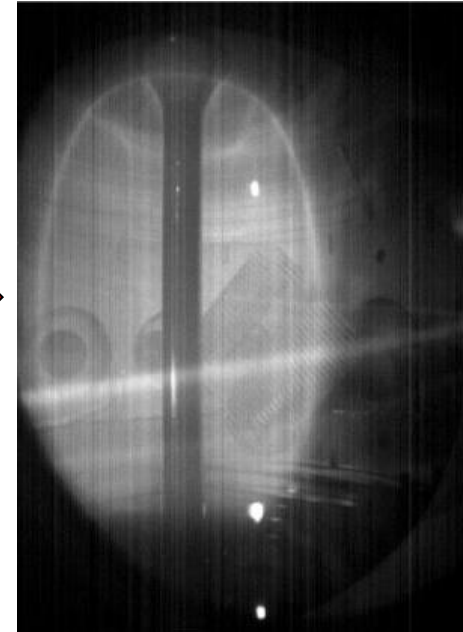
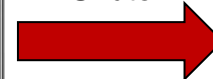
Null Formation



Relaxation



Injector  
Shutoff



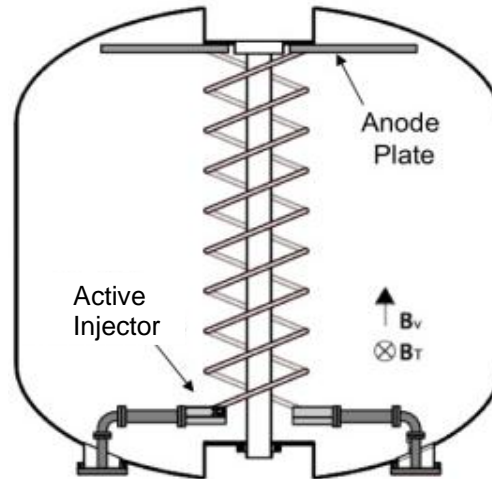
- Current injected along helical vacuum field
  - Local, active current sources
- MHD relaxation, tokamak-like state
  - Constrained by helicity, Taylor relaxation limits
- Tokamak plasmas produced after injector shut off
  - Couples to alternative current drive sources



# Helicity Input Provided by Edge-Localized Sources

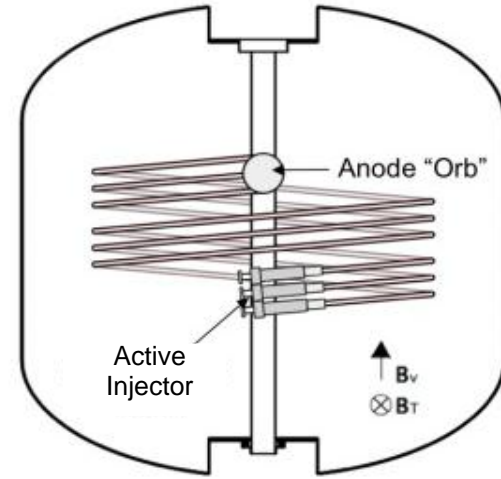
- Flexible injector geometry
- Active arc 'gun' injectors provide initial current windup, relaxation
- Either active guns or separate electrodes can provide further growth, sustainment

Inboard Injection\*

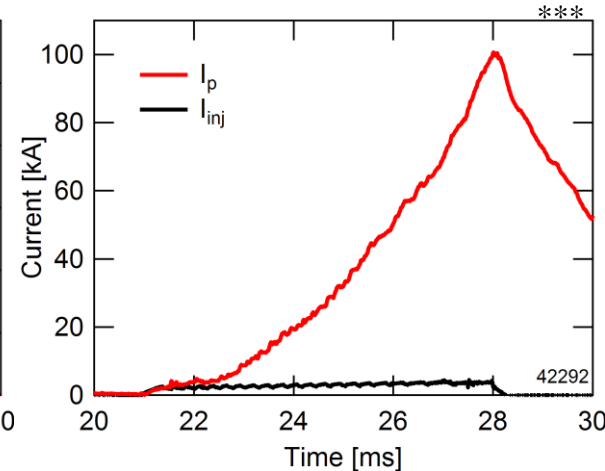
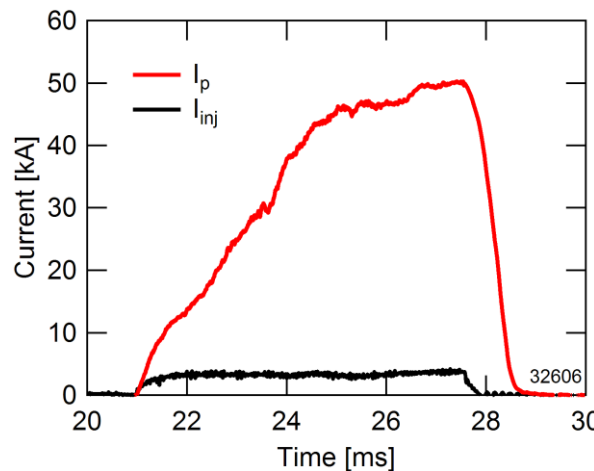


$$R_{inj} = 16 \text{ cm}, Z_{inj} = -75 \text{ cm}$$

Outboard Injection\*\*



$$R_{inj} = 70 \text{ cm}, Z_{inj} = -20 \text{ cm}$$







# Helicity Balance, Taylor Relaxation Criteria Determines Maximum Achievable $I_p$ from LHI

Helicity balance in a tokamak geometry:

$$\frac{dK}{dt} = -2 \int_V \eta \mathbf{J} \cdot \mathbf{B} d^3x - 2 \frac{d\psi}{dt} \Psi - 2 \int_A \Phi \mathbf{B} \cdot d\mathbf{s} \longrightarrow I_p \leq \frac{A_p}{2\pi R_0 \langle \eta \rangle} (V_{ind} + V_{eff})$$

- Helicity injection can be expressed as an effective loop voltage
- $I_p$  limit depends on plasma confinement via resistivity  $\eta$

$$V_{eff} \approx \frac{A_{inj} B_{\phi, inj}}{\Psi} V_{inj}$$

Taylor relaxation of a force-free equilibrium:

$$\begin{aligned} \nabla \times \mathbf{B} &= \mu_0 \mathbf{J} = \lambda \mathbf{B} \\ \lambda_p &\leq \lambda_{inj} \end{aligned} \longrightarrow \frac{\mu_0 I_p}{\Psi} \leq \frac{\mu_0 I_{inj}}{2\pi R_{inj} w B_{\theta, inj}}$$

$$I_p \leq \left[ \frac{C_p}{2\pi R_{inj} \mu_0} \frac{\Psi I_{inj}}{w} \right]^{1/2}$$

Assumptions:

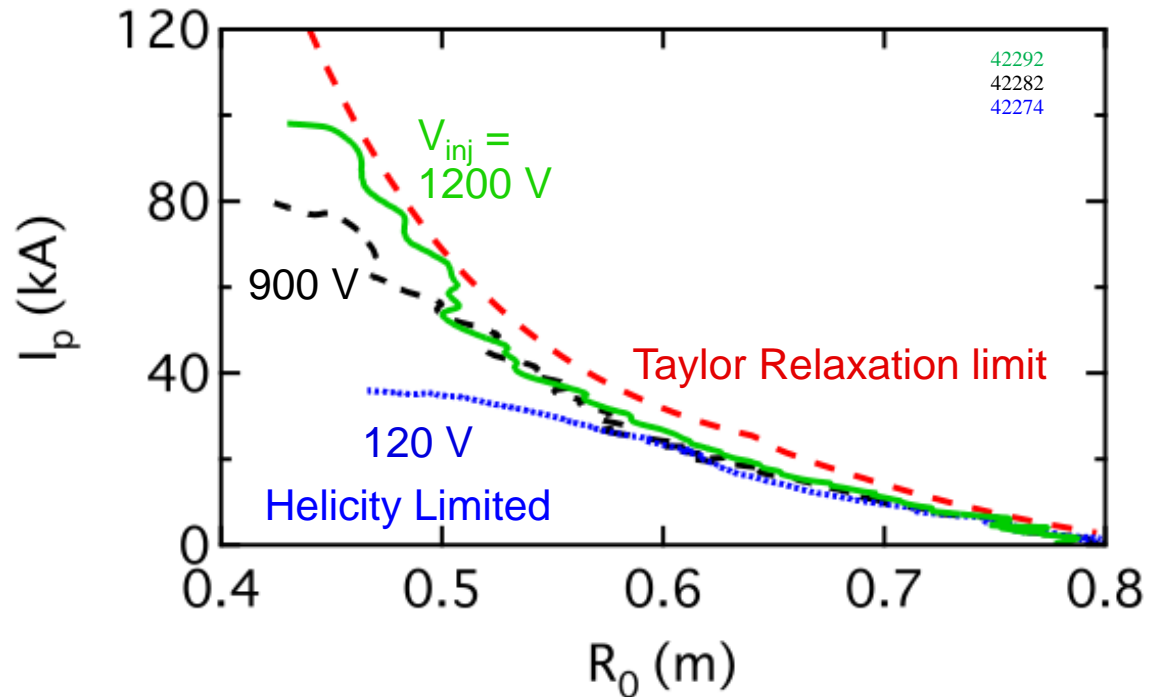
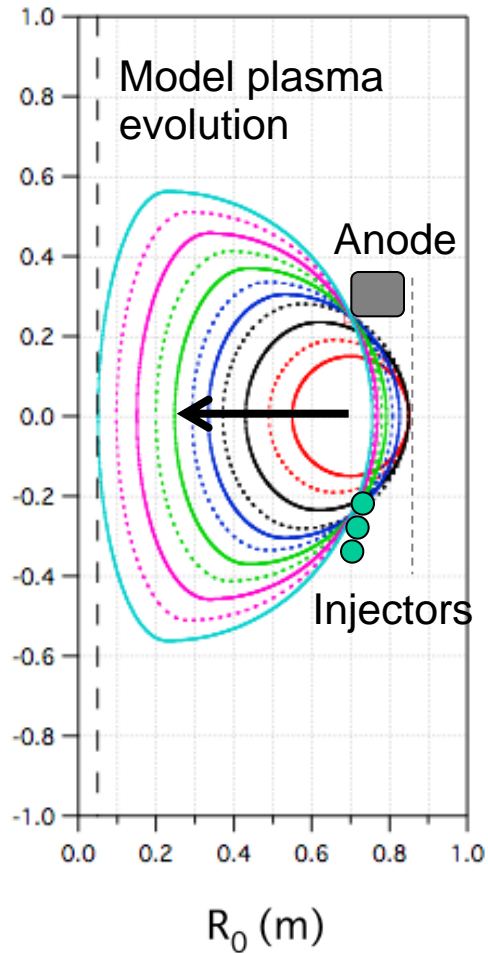
- Driven edge current mixes uniformly
- Edge fields average to tokamak-like structure

$A_p, A_{inj}$  : Plasma, injector area  
 $C_p$  : Plasma circumference  
 $\Psi$  : Plasma toroidal flux  
 $w$  : Edge current channel width





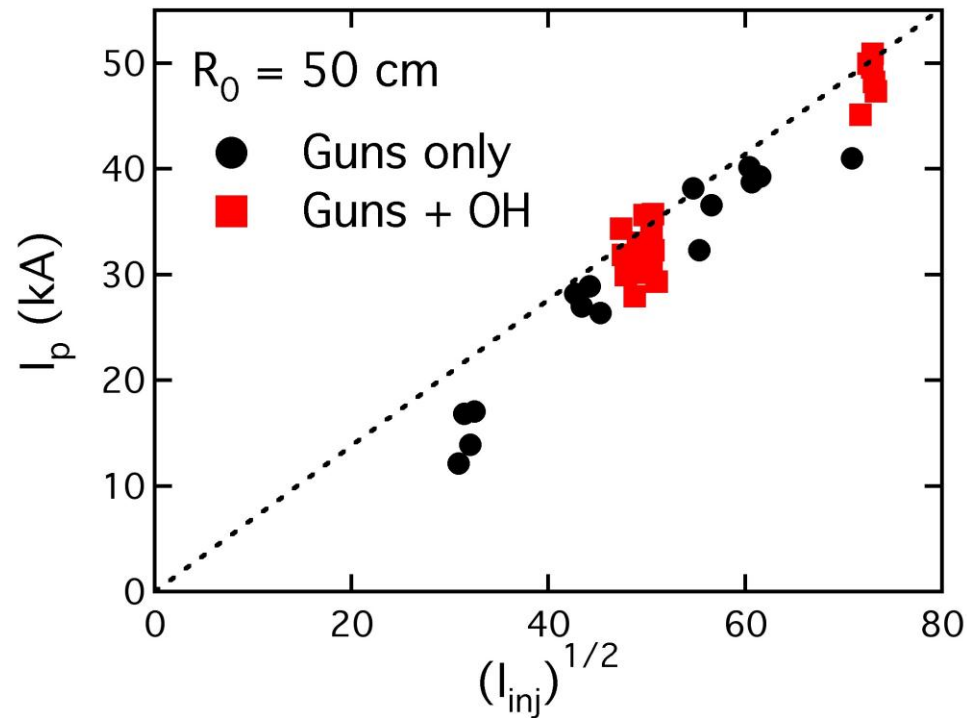
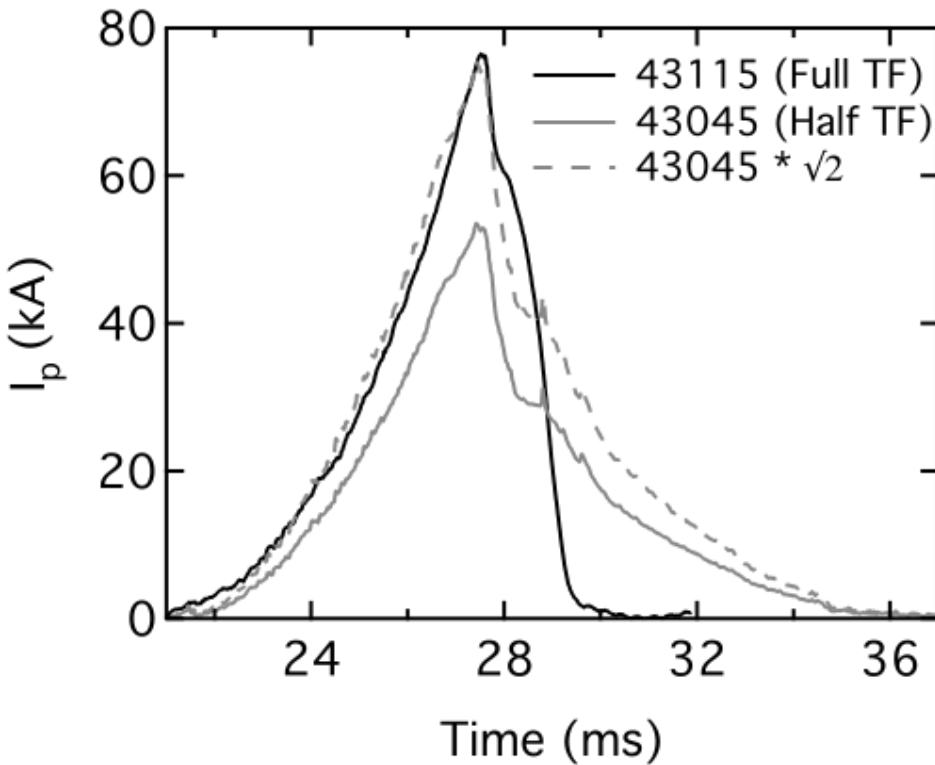
# Achieving the Maximum $I_p$ at the Taylor Limit Requires Sufficient Helicity Injection Input Rate



$$V_{eff} \approx \frac{A_{inj} B_{\phi, inj}}{\Psi} V_{inj}$$



# Experimental Plasma Currents Follow Taylor Limit Scalings



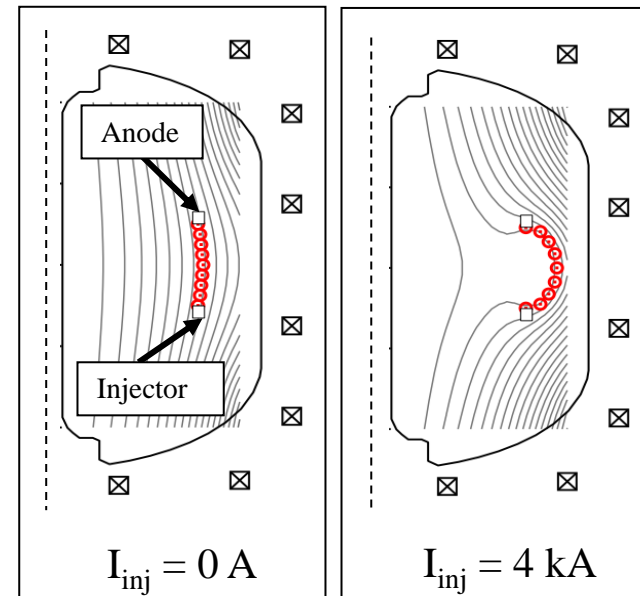
- Taylor limit:  $I_{p,max} \propto \sqrt{I_{TF} I_{inj}}$
- Limit appears absolute
  - Additional OH  $V_{loop}$  cannot raise  $I_p$  during LHI





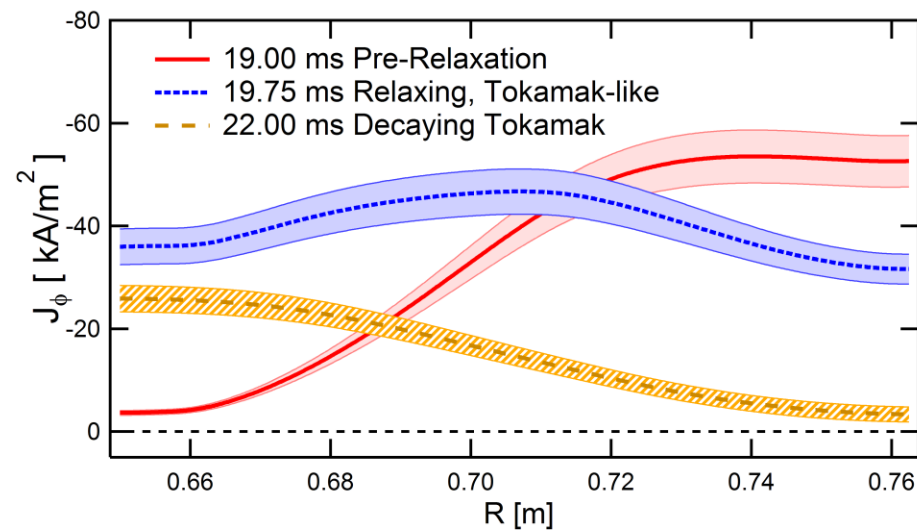
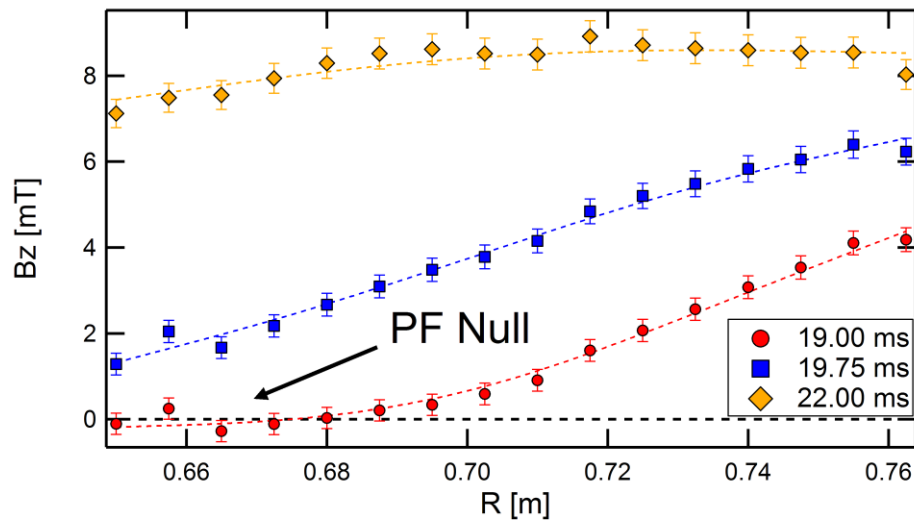
# Internal Measurements Show Null Formation, $J(R,t)$ Throughout LHI Discharge Evolution

- Initial relaxation to tokamak-like topology coincident with inboard null formation
  - Injected current filaments perturb vacuum  $\mathbf{B}$
  - $B_z$  must be sufficiently low and/or  $I_{inj}$  sufficiently high for null to form
- Hall probe\*  $B_z(R)$  provides  $J_\phi(R)$  evolution
  - Predicted field null observed



2-D force free current model

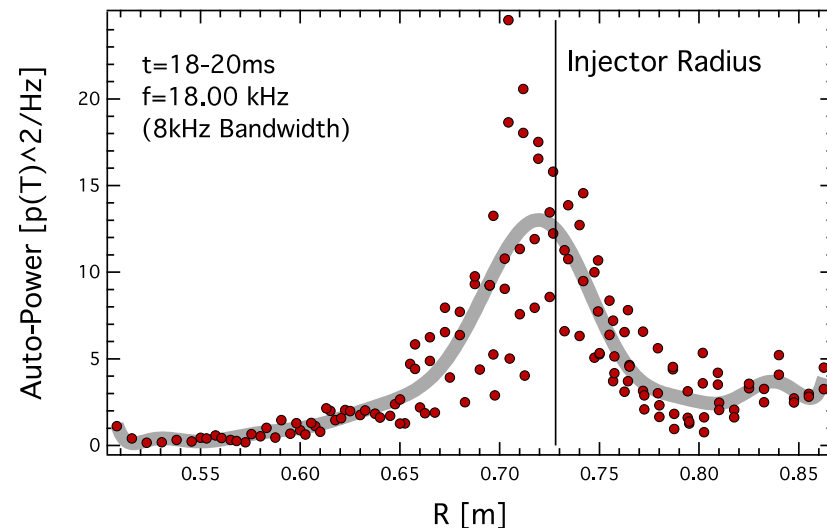
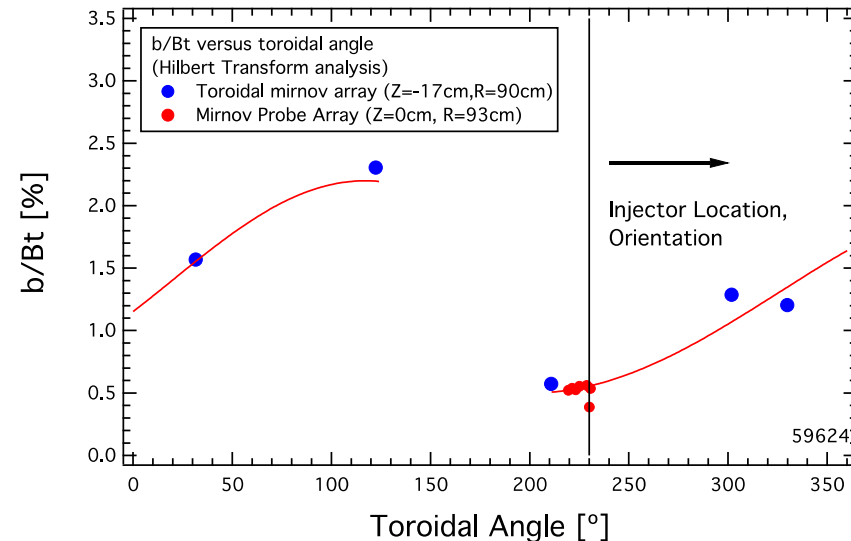
\*: Bongard *et al.*, Rev. Sci. Instrum. **81**, 10E105 (2010)





# Current Multiplication During LHI Accompanied by $n = 1$ Line-Tied Kink Activity

- Current multiplication, transport accompanied by MHD activity
- Two common spectral features
  - High-frequency 10–20 kHz  $n = 1$
  - Low-frequency  $< 5$  kHz  $n = 0$
- $n = 1$  mode consistent with line tying
  - Activity localized near injector radius
  - Toroidal asymmetry in  $\tilde{b}/B$
- $n = 0$  localized to plasma interior
  - Inward radial motion



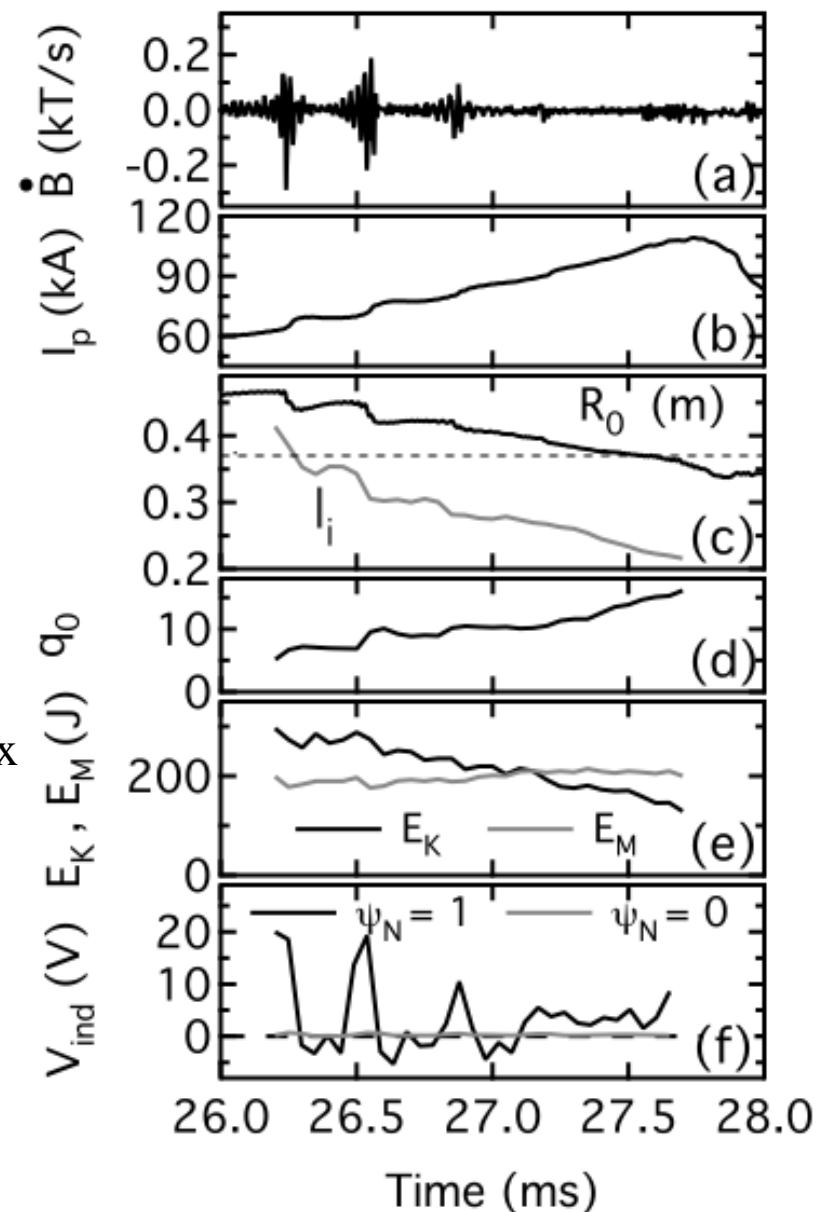


# Magnetic Topology Rapidly Changes with Bursts of MHD Activity During Helicity Injection

- Each burst typically  $\sim 0.1$  ms
- With each burst...
  - $\ell_i$  decreases  $\rightarrow I_p$  increases
  - $R_0$  decreases  $\rightarrow$  plasma expands
  - $B_{\phi,0}$  increases  $\rightarrow q_0$  increases
  - Slight drop in  $E_k$  and  $E_m$
  - Little change in poloidal flux at plasma edge
  - Rapid decrease in the total trapped poloidal flux
- Temporally and spatially averaged  $V_{ind} \sim 1.5$  V

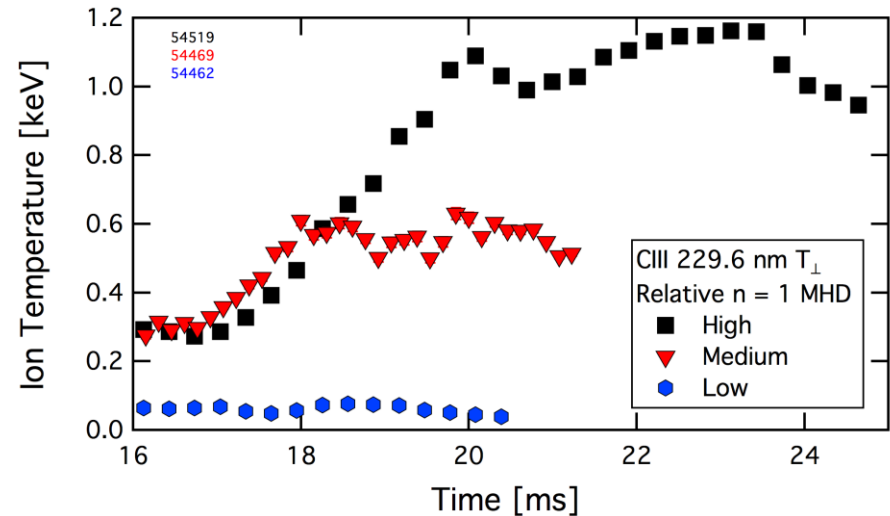
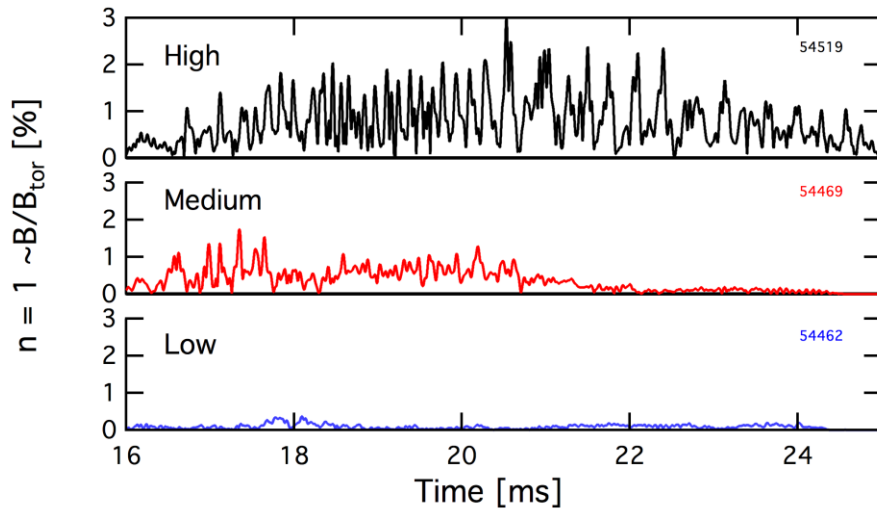
Battaglia *et al.*, Nucl. Fusion **51**, 073029 (2011)

M.W. Bongard, EPR Workshop, Fort Worth, TX Feb. 2013





# Strong, Anisotropic Ion Heating Observed During Helicity Injection



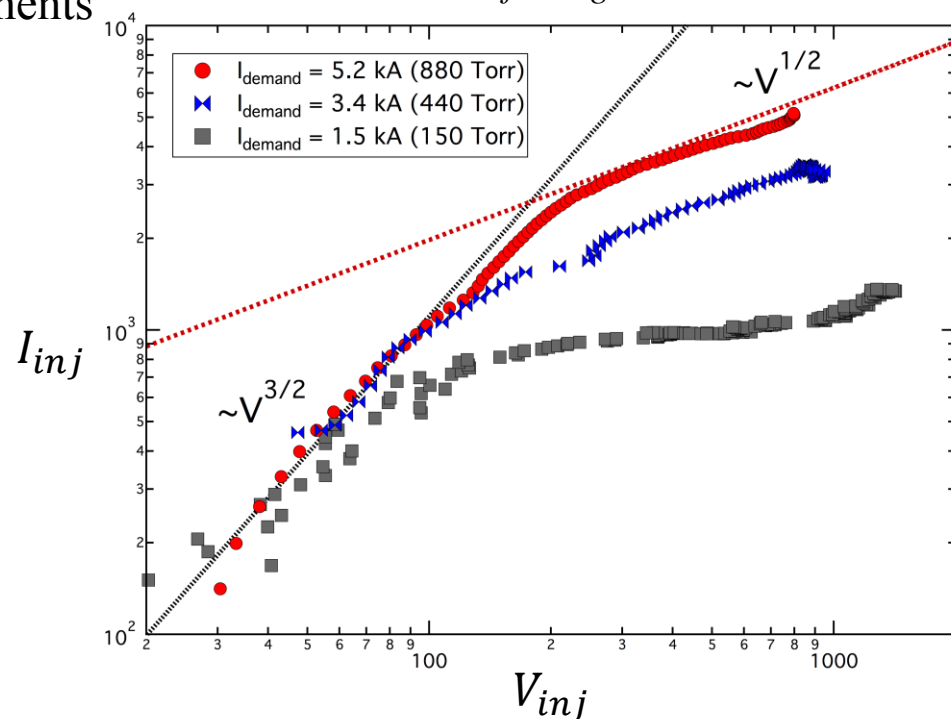
- Strong ion heating correlated with  $n = 1$  burst activity on multiple line species
- Ion  $T_{\perp} > 2 T_{\parallel}$  is often observed
  - Similar phenomenon observed in MST\*\* during magnetic reconnection



# Source Impedance Governed by Space Charge and Magnetic Current Limits

- Predictive impedance models required to design future startup systems
  - Taylor limit  $\propto \sqrt{I_{inj}}$  ; Helicity input  $\propto V_{inj}$
  - $Z_{inj}$  couples  $I_{inj}, V_{inj} \rightarrow$  power requirements
- Two distinct regimes evident in active source I-V characteristics
  - Double-sheath space-charge limit
    - Low  $I_{inj}, V_{inj}$
    - $I_{inj} \propto V^{3/2}$
  - Alfvén-Lawson magnetic current limit
    - High  $I_{inj}, V_{inj}$
    - $I_{inj} \propto V^{1/2}$
    - Sheath expansion may also contribute

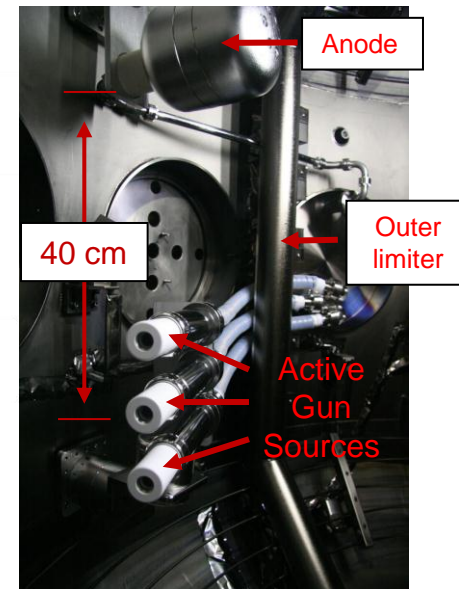
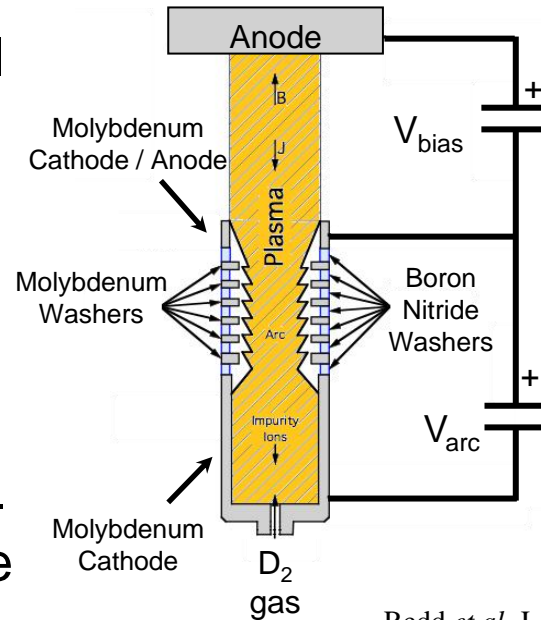
*I-V characteristics of arc plasma current injector for varied fueling rates.*





# Local Helicity Input Requires Increasingly Capable Electron Current Injectors

- Active gun sources used for initial relaxation, sustainment
  - Arc plasma created in coaxial washer gun
  - Electron current extracted from arc
- Subsequent growth via electrode-based systems may offer scalable path forward
  - Goal: simultaneously optimize helicity injection, Taylor relaxation constraints
    - High  $I_{inj}$  over extended area
- Need to develop large  $A_{inj}$  uniform current injector
  - Minimize gas load



Redd *et al*, J. Fusion Energy **28**, 203 (2009)



$A_{eff} \sim 6 \text{ cm}^2$



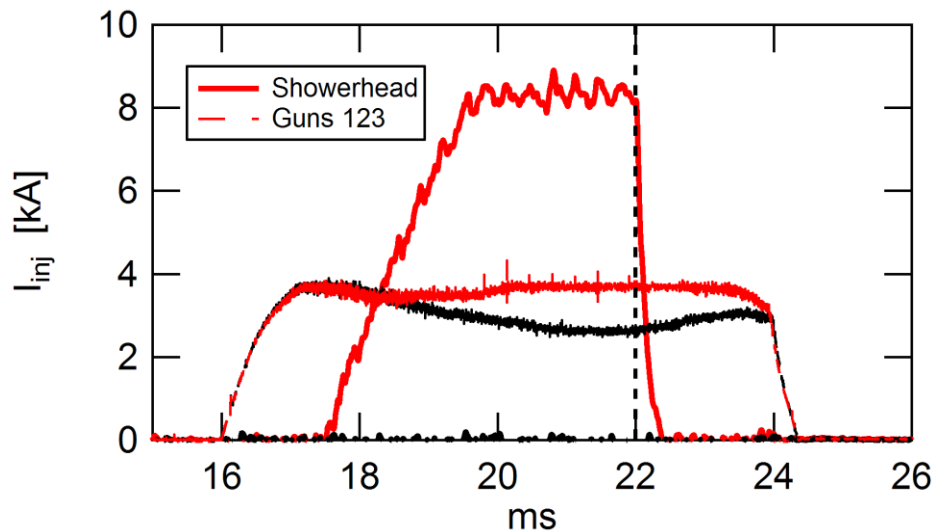
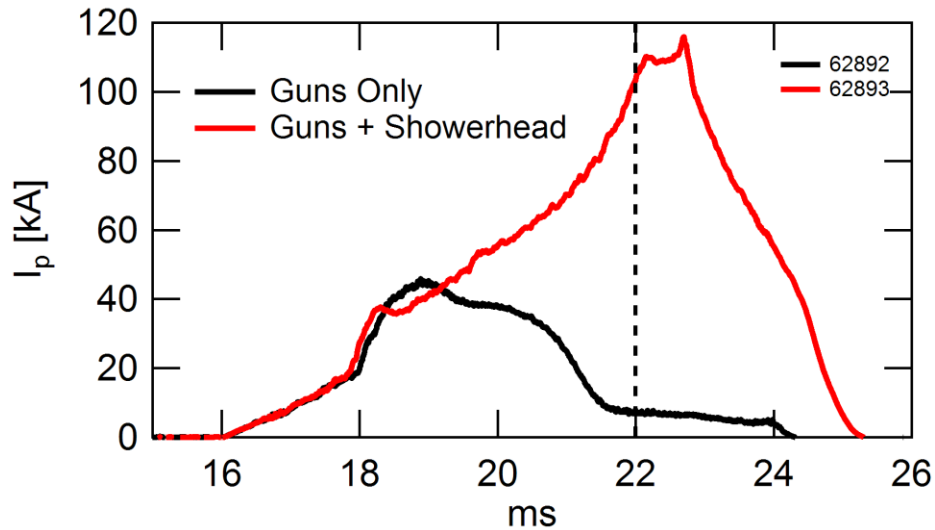
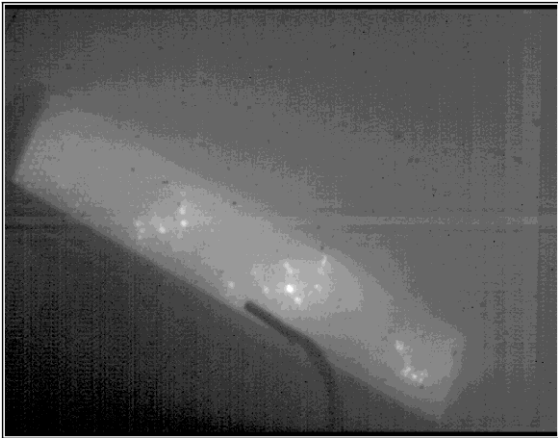
$A_{eff} \leq 50 \text{ cm}^2$





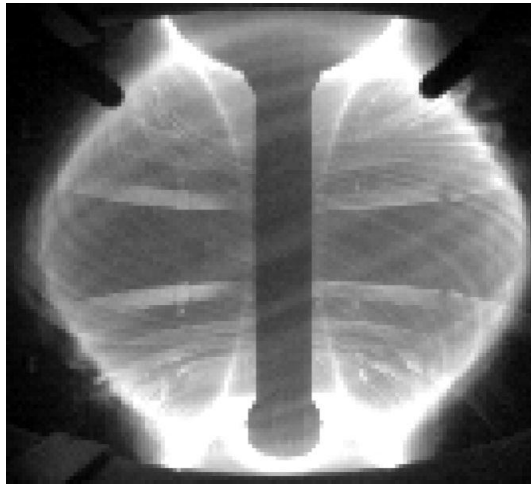
# New 'Showerhead' Electrode Designed for Hollow-Cathode, High Area Helicity Injection

- Promising results from initial commissioning of new electrode
  - $I_p > 100$  kA with showerhead assist;  
 $\leq 45$  kA without
  - Matched PF evolution, fueling
- Diffuse illumination of assembly,  $I_p$  increase suggests high  $A_{\text{eff}}$

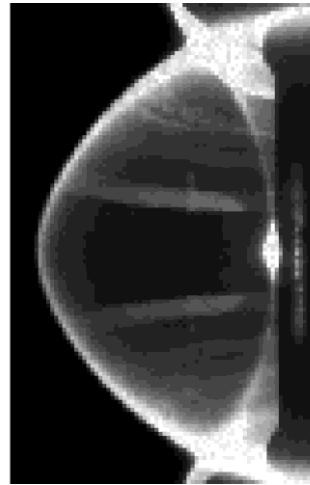




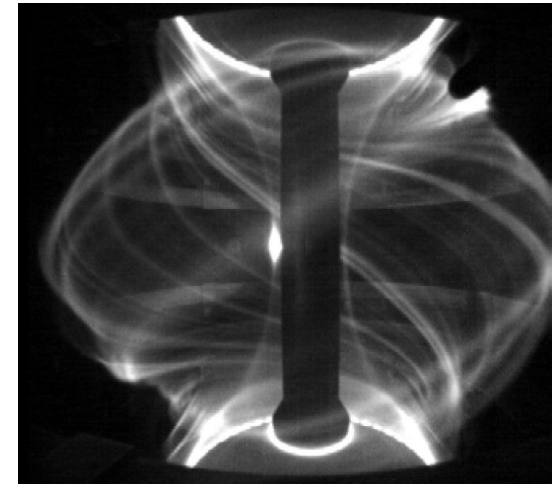
# Edge Stability Critical to Next-Step Fusion Devices



MAST L-Mode\*

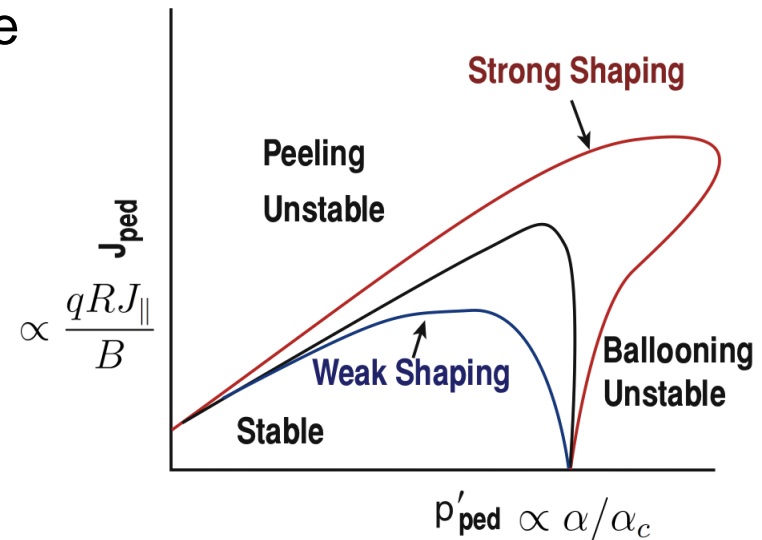


H-Mode\*



ELM\*\*

- Future fusion devices will operate in H-mode
  - Edge Localized Modes (ELMs) of concern
- Peeling-ballooning theory believed to underlie most damaging Type-I ELM
  - Pressure, current density gradients in edge drive ideal MHD instabilities
  - Detailed  $J_{\text{edge}}$  measurements needed



Stability Space\*\*\*

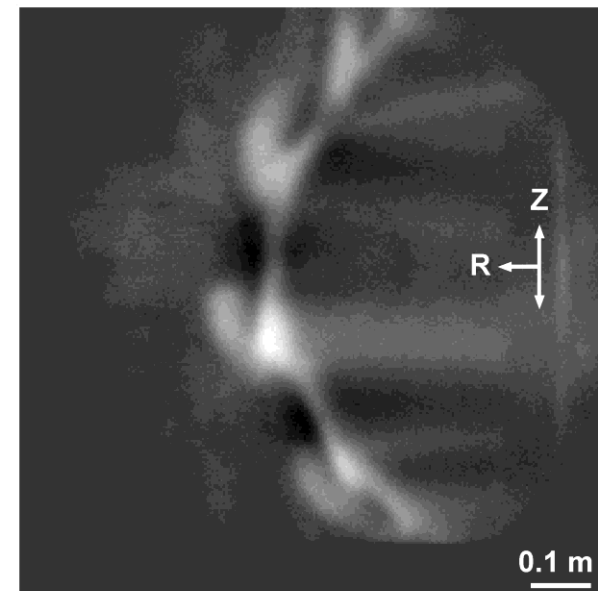
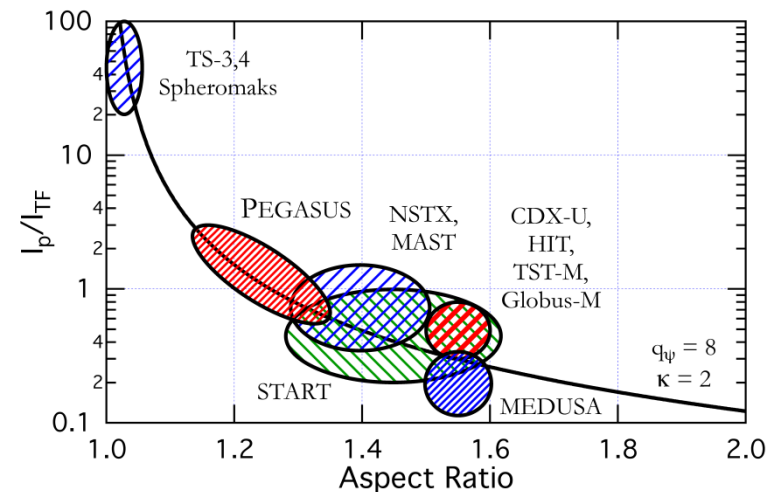
\*: Kirk *et al.*, Plasma Phys. Control. Fusion **48**, B433 (2006); \*\*: Kirk *et al.*, Plasma Phys. Control. Fusion **49**, 1259 (2007)

\*\*\*: Snyder, Phys. Plasmas **12**, 056115 (2005); Hegna, Phys. Plasmas **3**, 584 (1996)



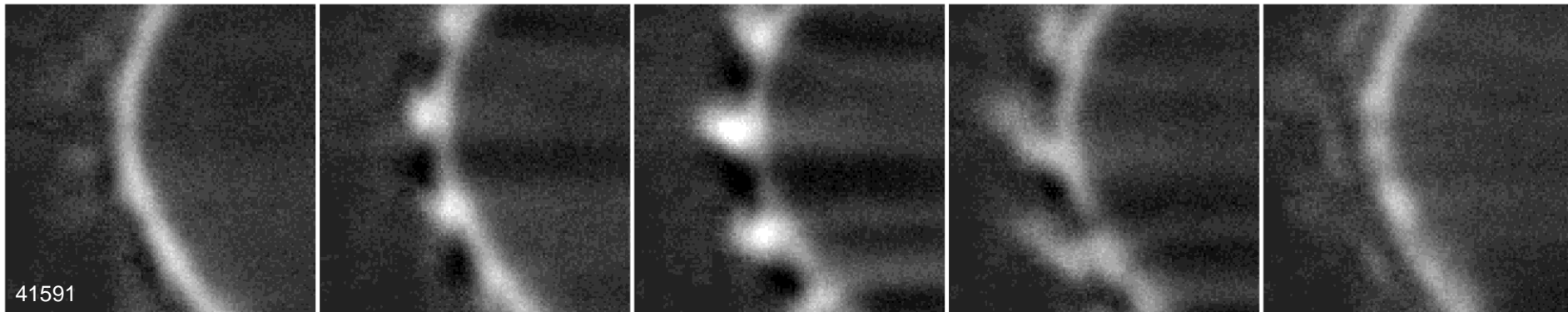
# Low-A PEGASUS ST Provides Access to Peeling Instability and Conditions to Measure J

- Spherical tokamaks naturally provide strong peeling drive
  - Toroidal field utilization  $I_p/I_{TF} \sim J_{||}/B$
- PEGASUS accesses peeling modes
  - Strong  $J_{||}/B \sim 1 \text{ MA/m}^2\text{-T}$  at  $A \leq 1.3$
  - Comparable to DIII-D in H-mode
- Machine parameters permit internal edge measurements
  - Short pulse lengths ( $< 50 \text{ ms}$ )
  - Modest  $T_e < 200 \text{ eV}$





# Pegasus Peeling Mode Features Match Empirical and Theoretical Expectations



0-9  $\mu\text{s}$

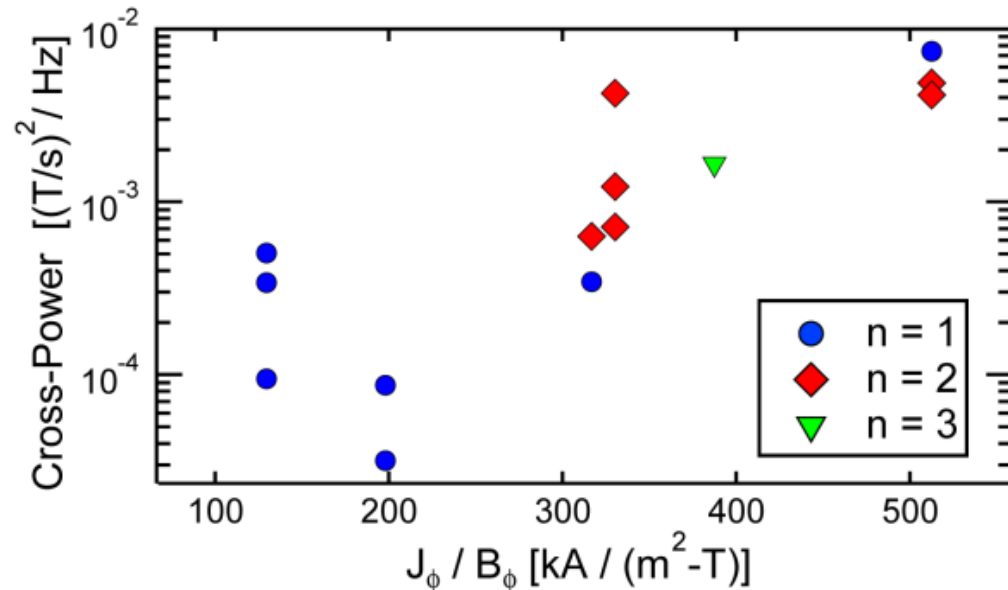
11-20  $\mu\text{s}$

22-31  $\mu\text{s}$

33-42  $\mu\text{s}$

44-53  $\mu\text{s}$

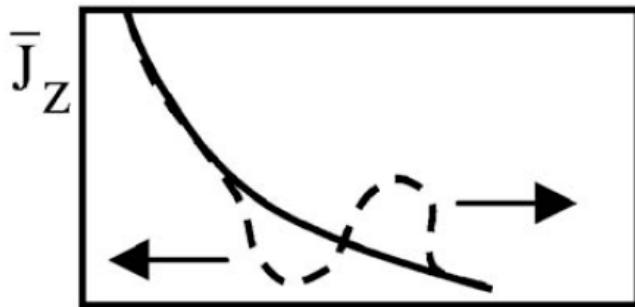
- Short lifetimes with high poloidal coherence
- Detachment, radial propagation of filaments
- High- $m$ , low- $n$  structure
- Mode amplitude increases with theoretical drive  $J/B$





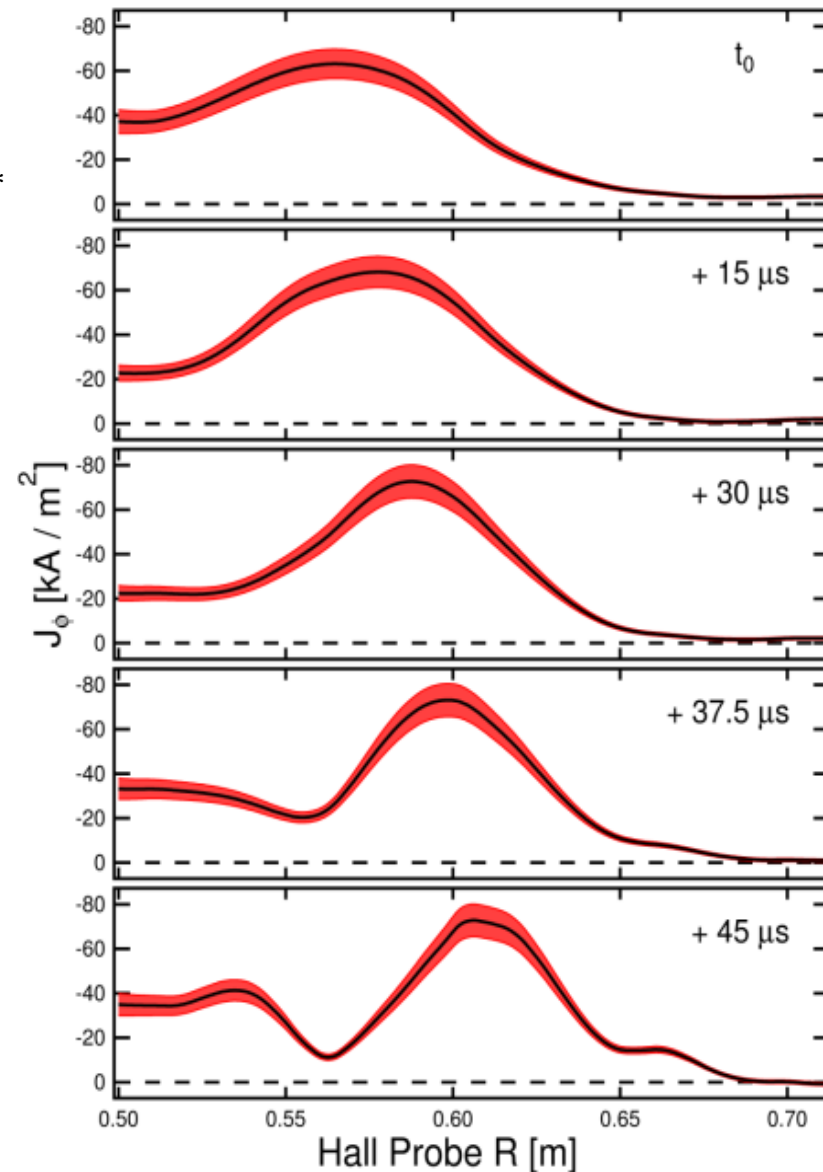
# $J_{\text{edge}}$ Dynamics Measured on ELM Timescales

- Peeling mode filament forms from initial “current-hole”  $J_{\text{edge}}$  perturbation\*
  - Validates formation mechanism hypothesized by EM blob transport theory\*\*
- Filaments carry current  $I_f \sim 100\text{-}220\text{ A}$ 
  - $I_f < 0.2\%$  of  $I_p$ , similar to MAST ELMs
- Radial motion qualitatively consistent with transient magnetostatic repulsion
  - Measured  $v_R$  consistent with available analytic models\*\*\*



\*\* : Myra, Phys. Plasmas **14**, 102314 (2007)

\*\*\* : Myra *et al.*, Phys. Plasmas **12**, 092511 (2005)



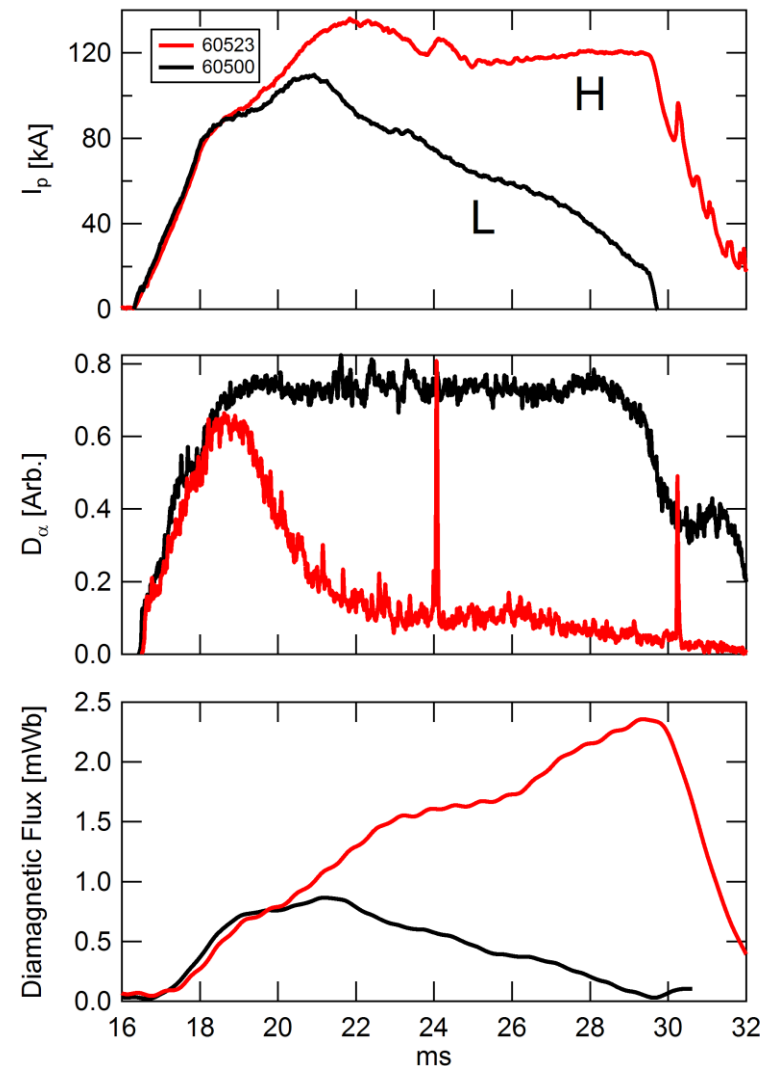
\* : Bongard *et al.*, Phys. Rev. Lett. **107**, 035003 (2011)





# H-mode Access: More Detailed ELM Tests and Possible Post-HI Current Drive Enhancement

- Ohmic H-mode achieved with new central column (high-field-side) fueling system
  - Standard L-mode with strong low-field-side external fueling
- Standard H-mode signals seen
  - Reduced  $D_\alpha$  emission
  - Quiescent edge between ELM events
  - Type I and III ELMs suggested
  - Improved confinement inferred

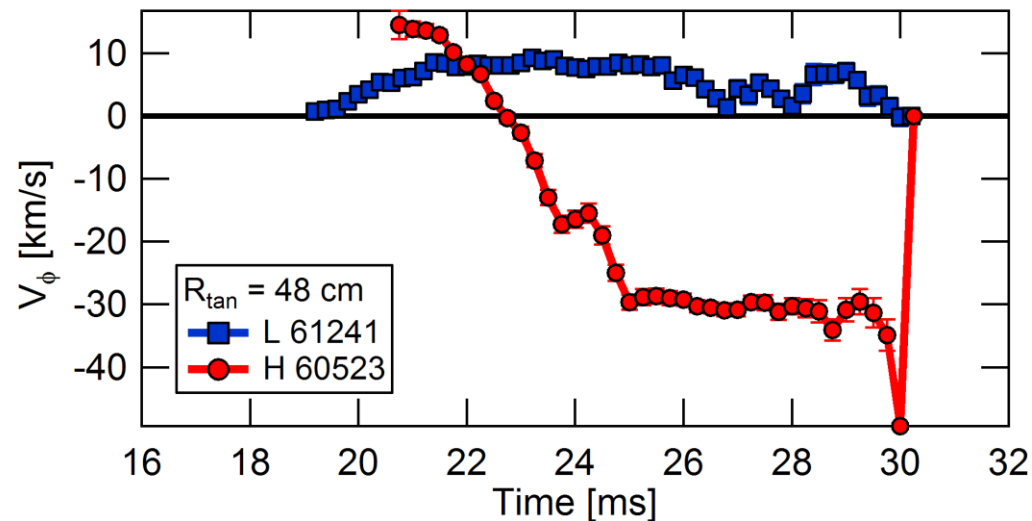






# Toroidal Flow Reverses at L–H Transition

- Toroidal rotation measured via  $T_i$  spectrometry\* in L, Ohmic H-mode discharges
  - No external momentum input
- L-mode flows are in the counter-current direction
- H-mode shots reverse rotation at  $L \rightarrow H$  transition
  - Effect seen on MAST\*\* and NSTX during HFS fueling



\*: Burke, *et al.*, Rev. Sci. Instrum. **83**, 10D516 (2012)

\*\* : Meyer *et al.*, J. Phys.: Conf. Ser. **123**, 012005 (2008)



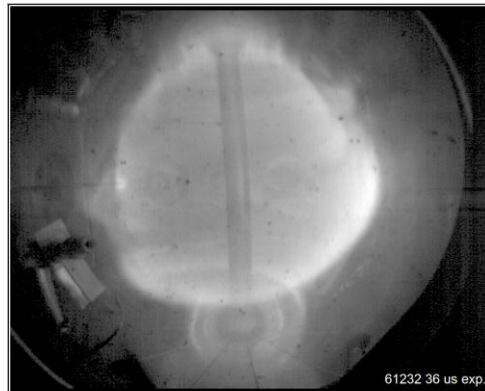
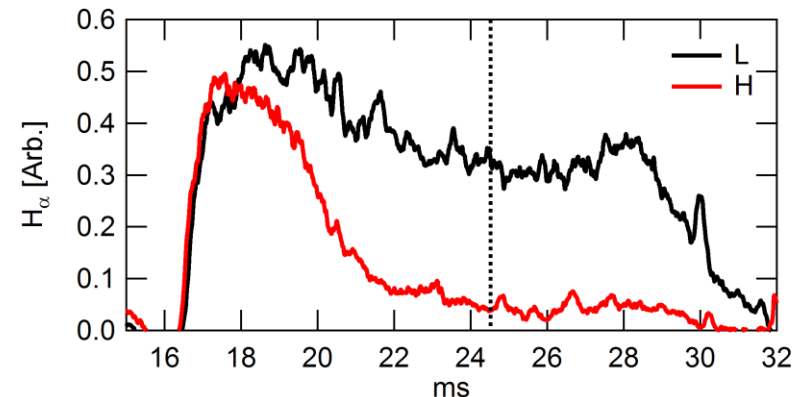
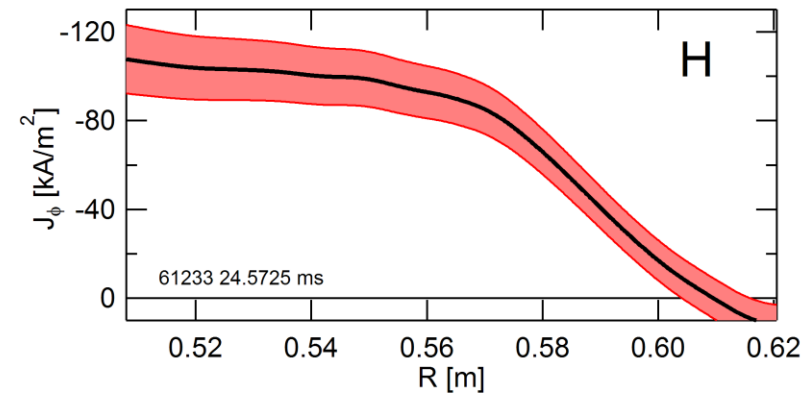
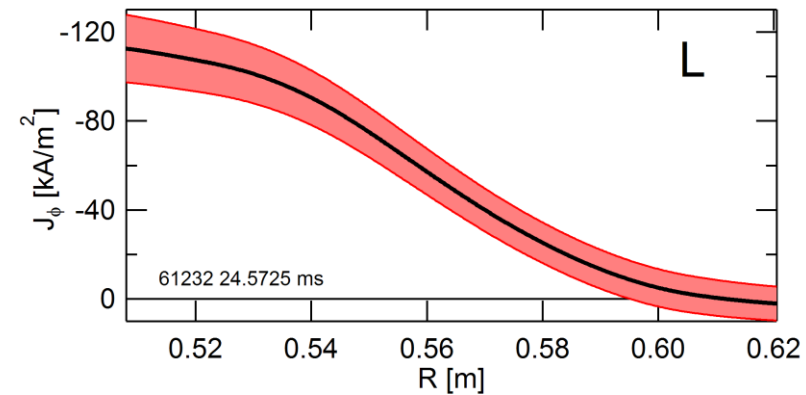


# Edge Current Pedestal Observed in H-Mode

- Internal B measurements from Hall array\* yield local  $J_\phi(R,t)$ \*\*
- Current gradient scale length significantly reduced in H-mode
  - L  $\rightarrow$  H: 6  $\rightarrow$  2 cm

\*: M.W. Bongard *et al.*, Rev. Sci. Instrum. **81**, 10E105 (2010)

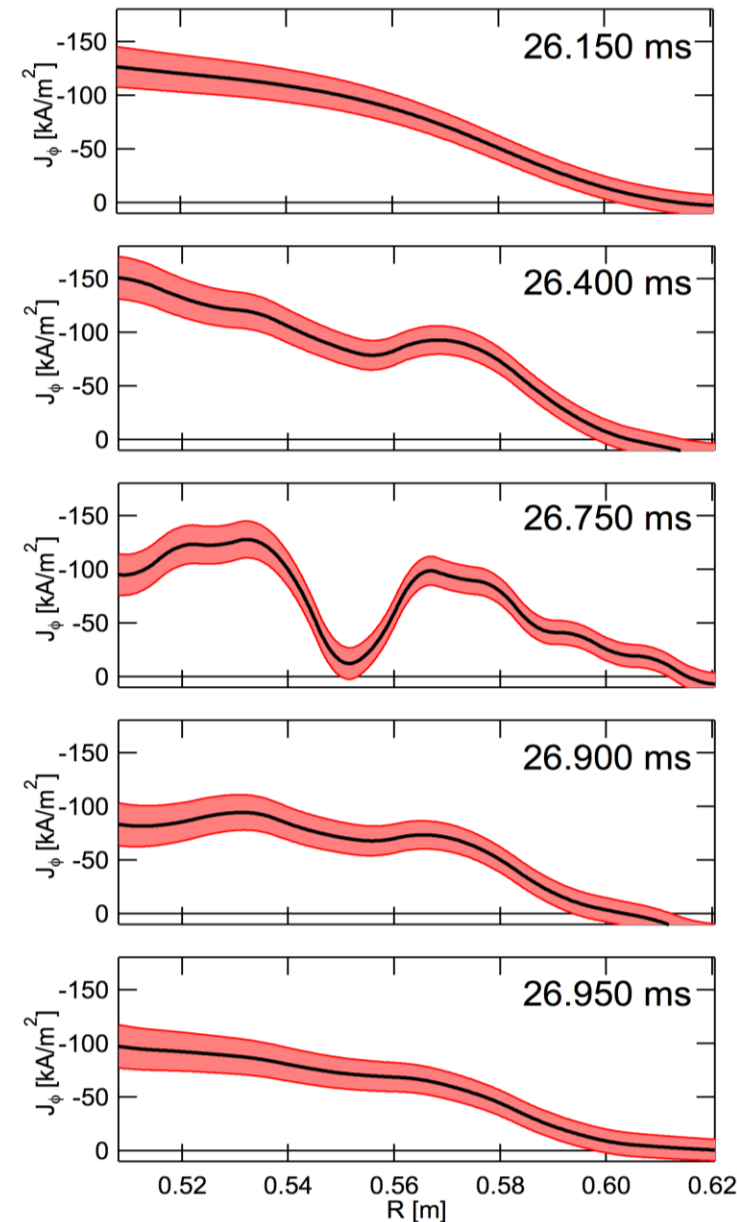
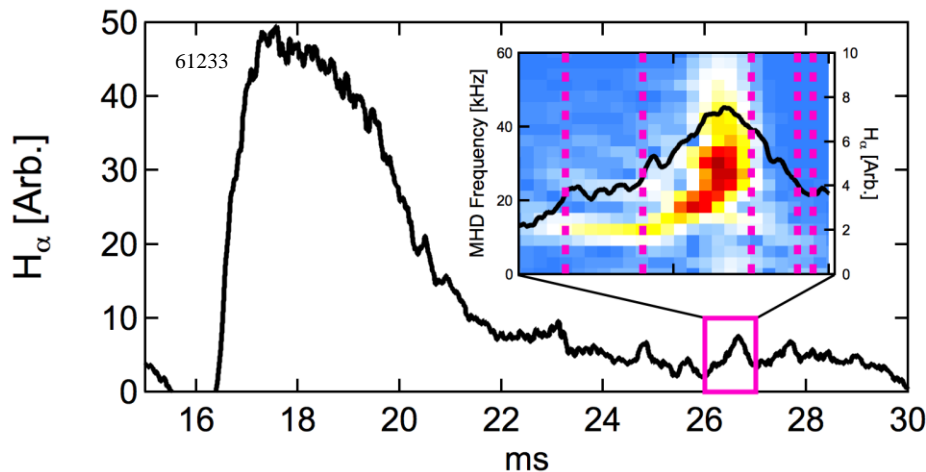
\*\* : C.C. Petty *et al.*, Nucl. Fusion **42**, 1124 (2002)





# $J_{\text{edge}}$ ELM Dynamics Observed

- $J(R,t)$  profiles measured throughout single Type III ELM
  - $n = 1$  EM precursor
  - $\sim 10\%$   $I_p$  loss, negligible  $\Delta\Phi$
- Current-hole perturbation accompanies pedestal crash
  - Similar to peeling modes in Pegasus
- Rapid recovery of H-mode pedestal





# Studies at Near-Unity Aspect Ratio Advance Fusion Energy Sciences

- Significant progress with non-solenoidal startup of ST
  - Increasing understanding of HI physics to project towards MA-class startup
    - Helicity balance, relaxation current limits determine ultimate  $I_p$
    - Complex MHD drives  $J(R,t)$  and reconnection-driven ion heating
    - Sheath and magnetic current limits govern injector impedance
  - Developing advanced edge current sources for increased helicity injection
- Leveraging low-A regime to test edge stability theory
  - Peeling mode characteristics consistent with theory
    - Onset, spatial structure, MHD virulence consistent with ideal MHD
    - Nonlinear dynamics: filament creation / propagation from  $J_{\text{edge}}$  current-hole
  - ITER-relevant ELM stability tests of peeling-ballooning modes
- LHI  $J(R,t)$  control and H-mode access support high- $\beta$  studies of tokamak limits
  - Deploying enhanced divertor coils for separatrix operation

