



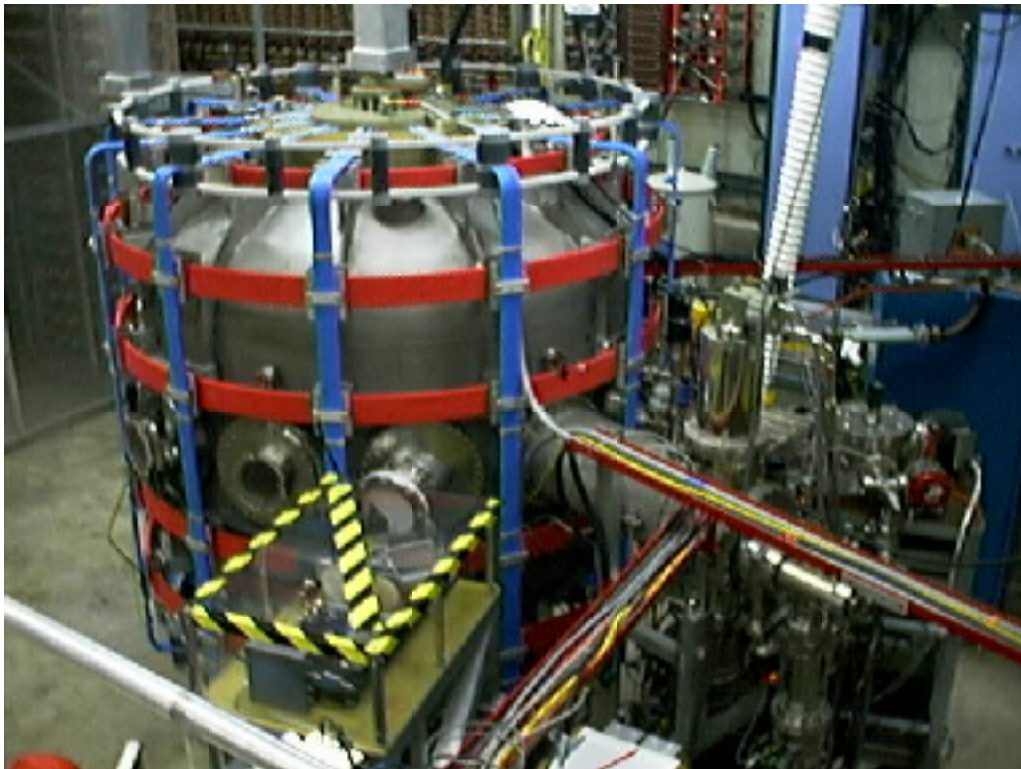
Recent Results from the PEGASUS Toroidal Experiment

G.D. Garstka

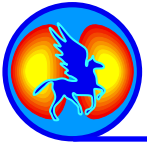
for the PEGASUS Group

University of Wisconsin–Madison U.S.A.

An extremely low-aspect ratio facility exploring quasi-spherical high-pressure plasmas with the goal of minimizing the central column while maintaining good confinement and stability.



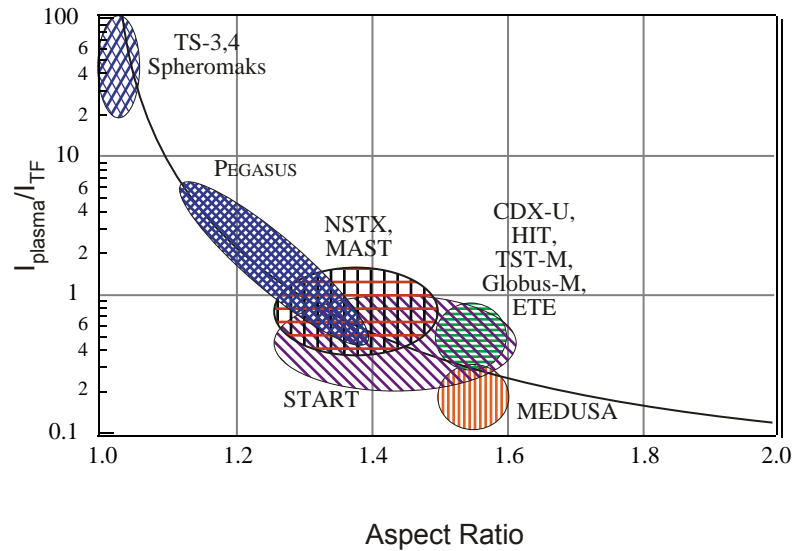
*Supported by U.S. DoE grant No. DE-FG02-96ER54375



Role of the PEGASUS Experiment

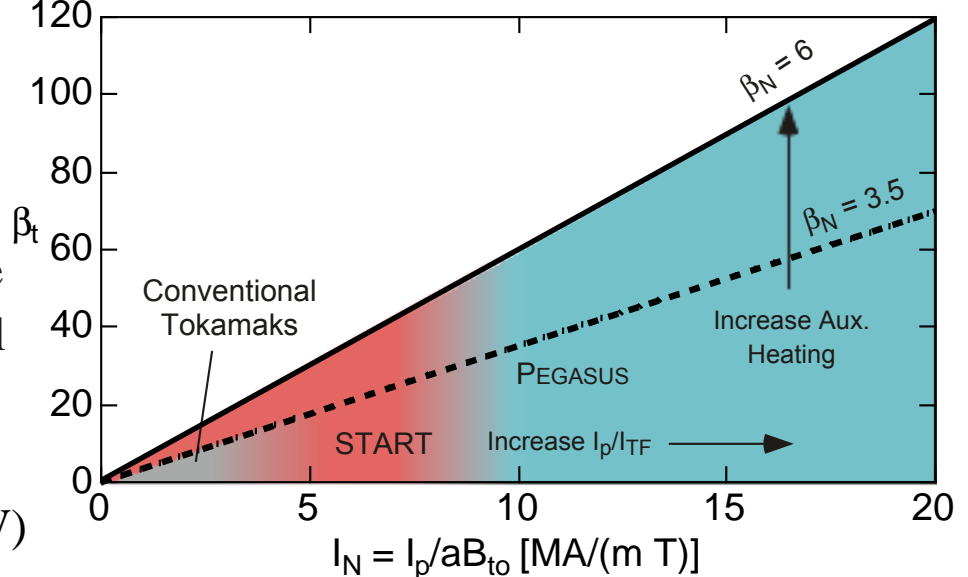
• Physics of $A \rightarrow 1$ plasmas as an Alternate Concept (low q)

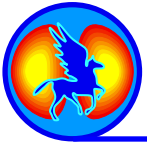
- Extreme toroidicity ($A \rightarrow 1$)
- Very high TF utilization (I_p/I_{TF}) > 3
- Stability at very low TF ($\beta \ll 1$)
- Relaxation stability at tokamak/spheromak boundary
- RF heating and CD schemes (HHFW, EBW)
- Trade-offs: CD, recirculating power, and $A \rightarrow 1$, low-TF operation



• Contribute to development of the ST (high q)

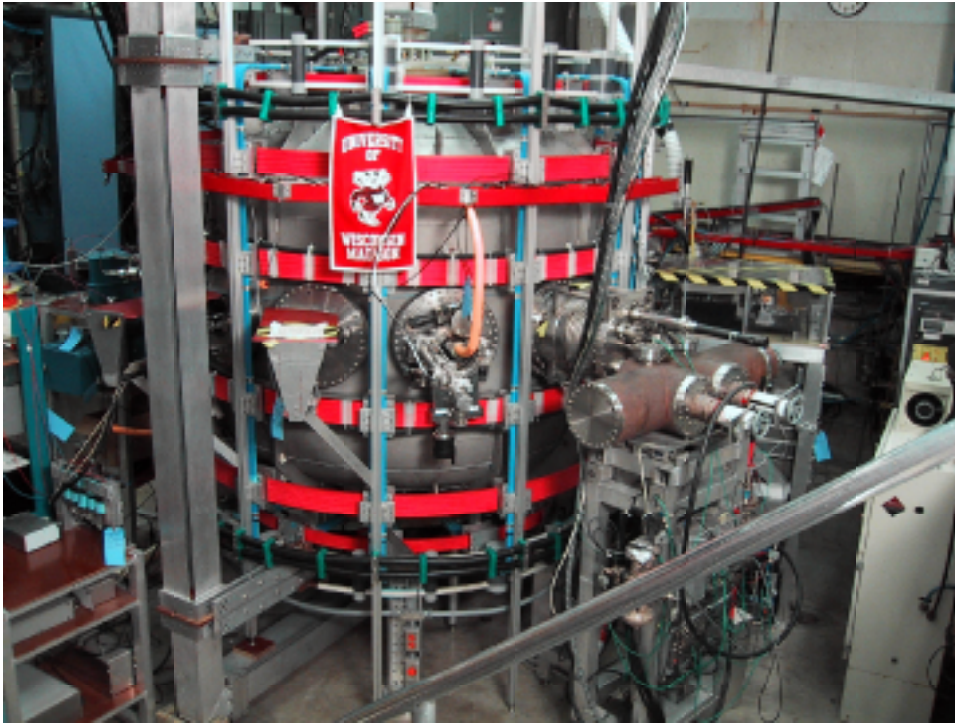
- Stability limits for $A \rightarrow 1$ (vs. I_p/I_{TF} , q_ψ , N_e , β_t , β_{pol} , κ , A , etc.)
- β limit dependencies
- Access high β_t at extreme I_N w/o conducting shell
- Confinement $A < 1.3$
- New startup schemes (e.g., plasma gun, EBW)





Pegasus is a Mid-Sized University ST

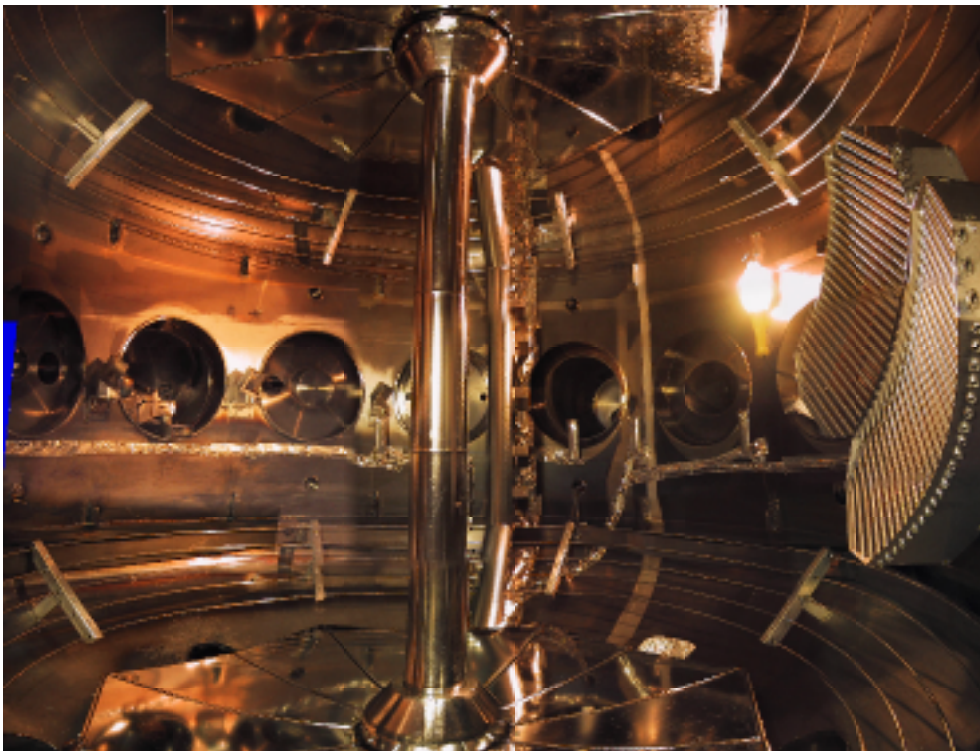
- High-Stress Solenoid Allows Exploration of $A \rightarrow 1$ Regime



Design Parameters

A	1.1 - 2.0
R	0.2 - 0.45 m
I_p	0.1 - 0.3 MA
B_t	< 0.30 T
K	~ 1.5 - 3.7
Δt_{pulse}	30 - 40 msec
β_t	O(1)
β_N	> 5
I_N	> 10
Heating and Sustainment	Inductive* + RFCD (HHFW, EBW)

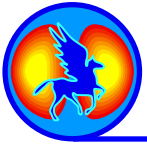
* NHMFL: $B_{\text{solenoid}} = 10 - 14$ T





Mid 2001: Status of Ohmic Plasma Operation

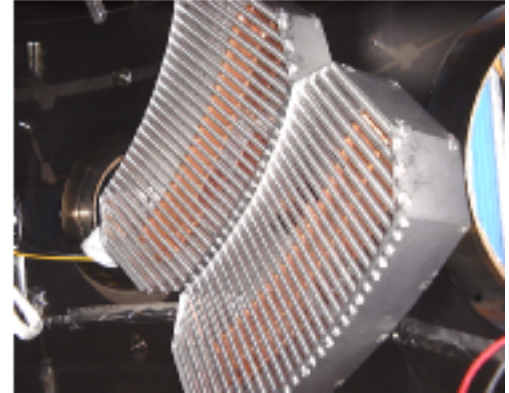
- **Routine high-stress solenoid operation**
- **Startup at low B_t in presence of conducting walls**
 - Induced wall currents modeled
 - Wall currents routinely included in equilibrium runs
- **Plasmas show low-A characteristics**
 - *High β_t* $\beta_t \sim 25\%$
 - *High β_N* $\beta_N \sim 5$
 - *High TF utilization factor* $I_p/I_{TF} \sim 1.2$
 - *High normalized current* $I_N \sim 8$
 - *High density* $n_e \sim n_{GW}$
 - *MHD* $2/1$, IREs, double tearing modes
- **Extension of operating space**
 - Increasing ohmic drive capability; I_p up to 140 kA
 - Density control and fueling (fast gas puffing)
 - Pulse length extension (up to 30 ms)
 - Wall conditioning (Ti gettering, DC GDC)
- **Major upgrades**
 - Extensive internal magnetics
 - New plasma-facing components
 - HHFW antenna installed and tested up to 100 kW
 - Several new diagnostics coming online



Major Machine Modifications to Upgrade Internal Hardware

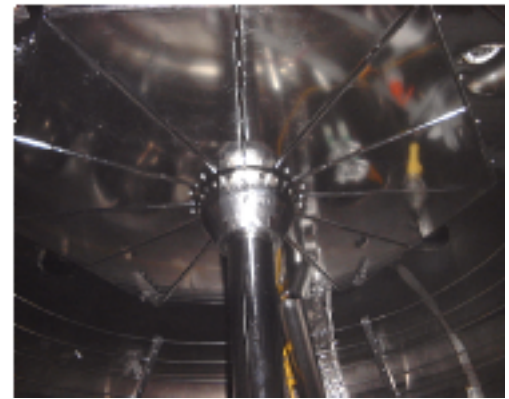
- **Extensive internal diagnostics installed**

- Flux loop and B_{pol} arrays
- Extensive centerstack magnetics
- Diamagnetic loop and new Rogowski coils



- **HHFW and EBW antennae installed**

- $P_{RF} = 1-2$ MW available for HHFW
- Fully steerable EBW/ECH antenna

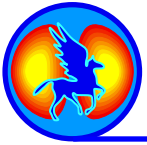


- **Improved power deposition hardware**

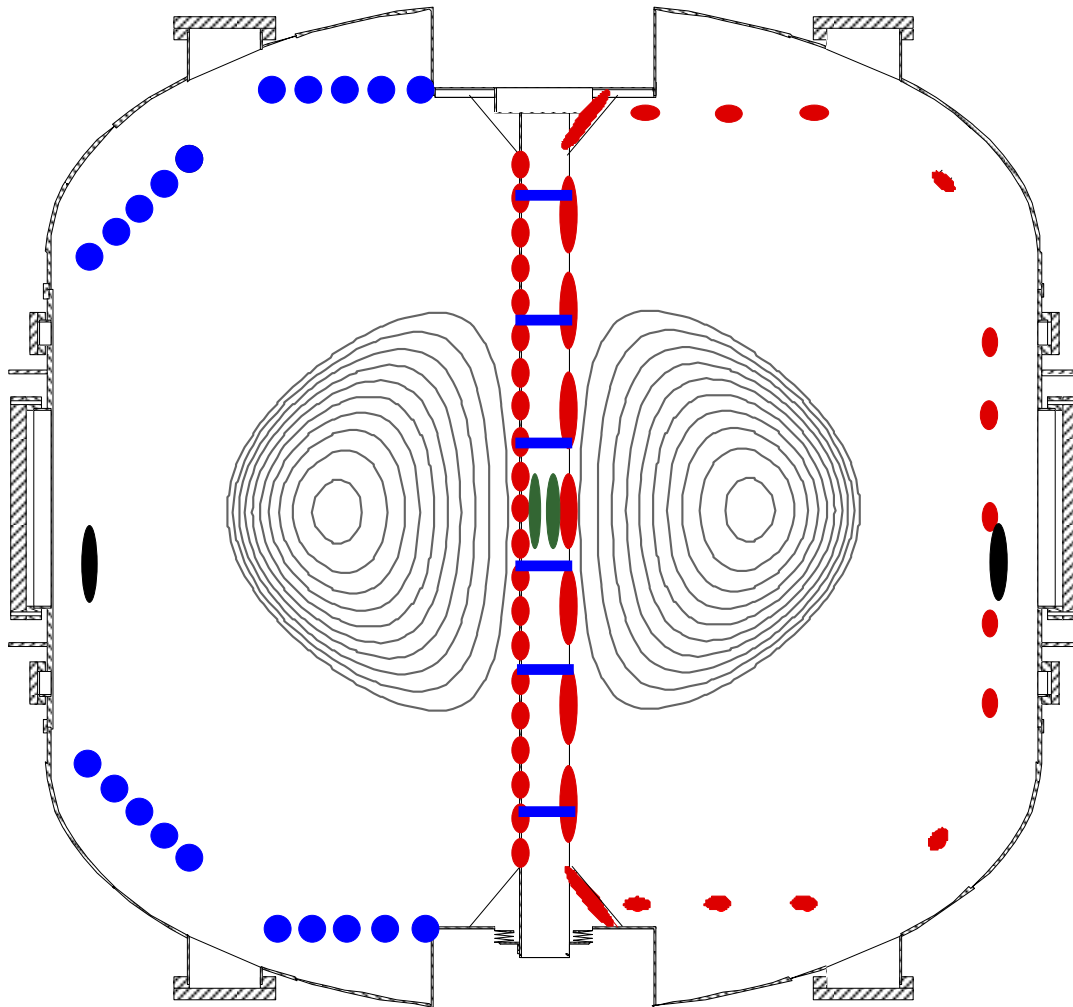
- Divertor plates
- High-power outer limiter

- **Near-Future Modifications to further improve performance**

- New toroidal field center rod assembly
 - Low inductance 12-turn \Rightarrow rapid B_t decrease during shot*
 - Increased TF, up to 0.3T \Rightarrow increased startup stability*
- Increased Volt-seconds from high-stress solenoid
 - Power supply improvements*
- Commission high power HHFW system for heating



New magnetics diagnostics installed in 2001



● Flux Loops (26)

● Poloidal Mirnov Coils (22 + 21)

● LFS Toroidal Mirnov Coils (6)

● HFS Toroidal Mirnov Coils (7)

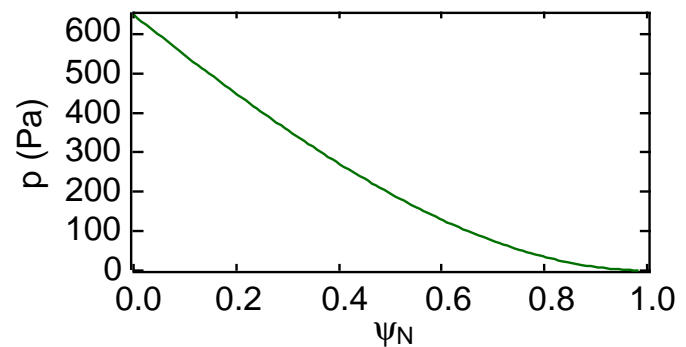
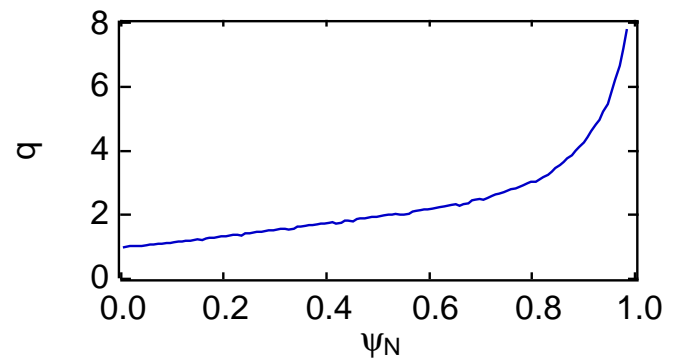
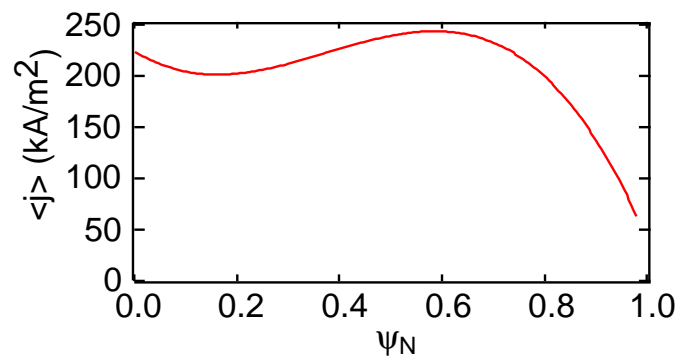
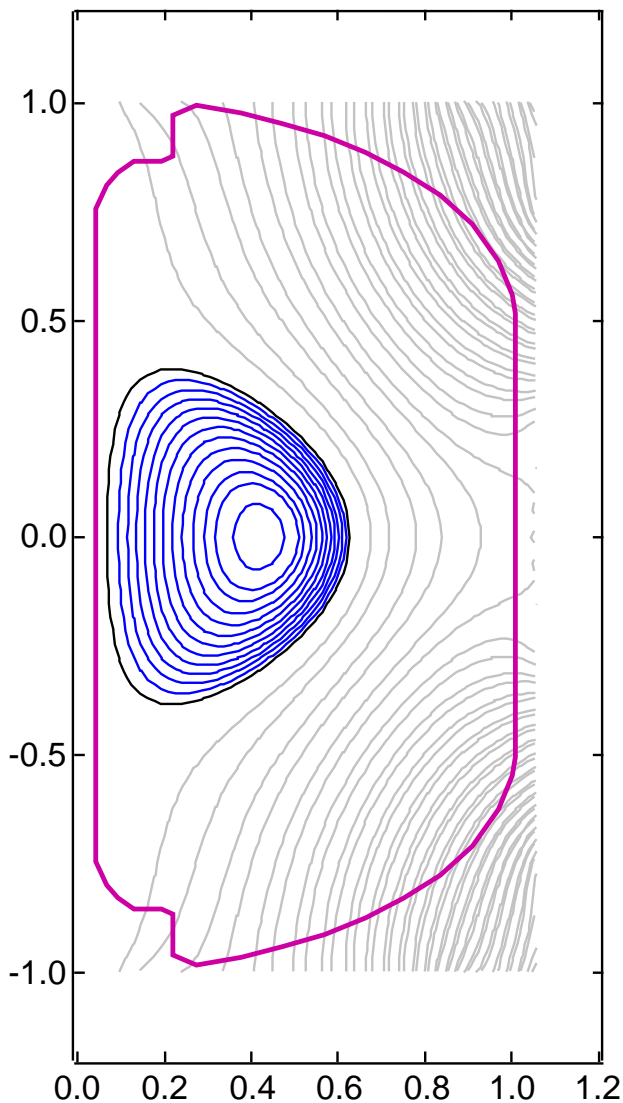
Not shown:

- Plasma Rogowski Coils (2)
- Diamagnetic Loops (2)
- Diamagnetic Compensation Loop
- External Wall Loops (6) [constrain wall currents]
- Internal B_{tan} Coils (15) [constrain wall currents]

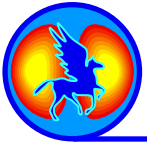


Magnetic Equilibrium Reconstruction Used as Primary Analysis Tool

Shot	I_p	78.3 kA	B_t (axis)	0.048 T
12445	R_0	0.337 m	β_t	18%
	a	0.282 m	l_i	0.40
	A	1.20	q_0	0.98
	κ	1.4	q_{98}	7.8

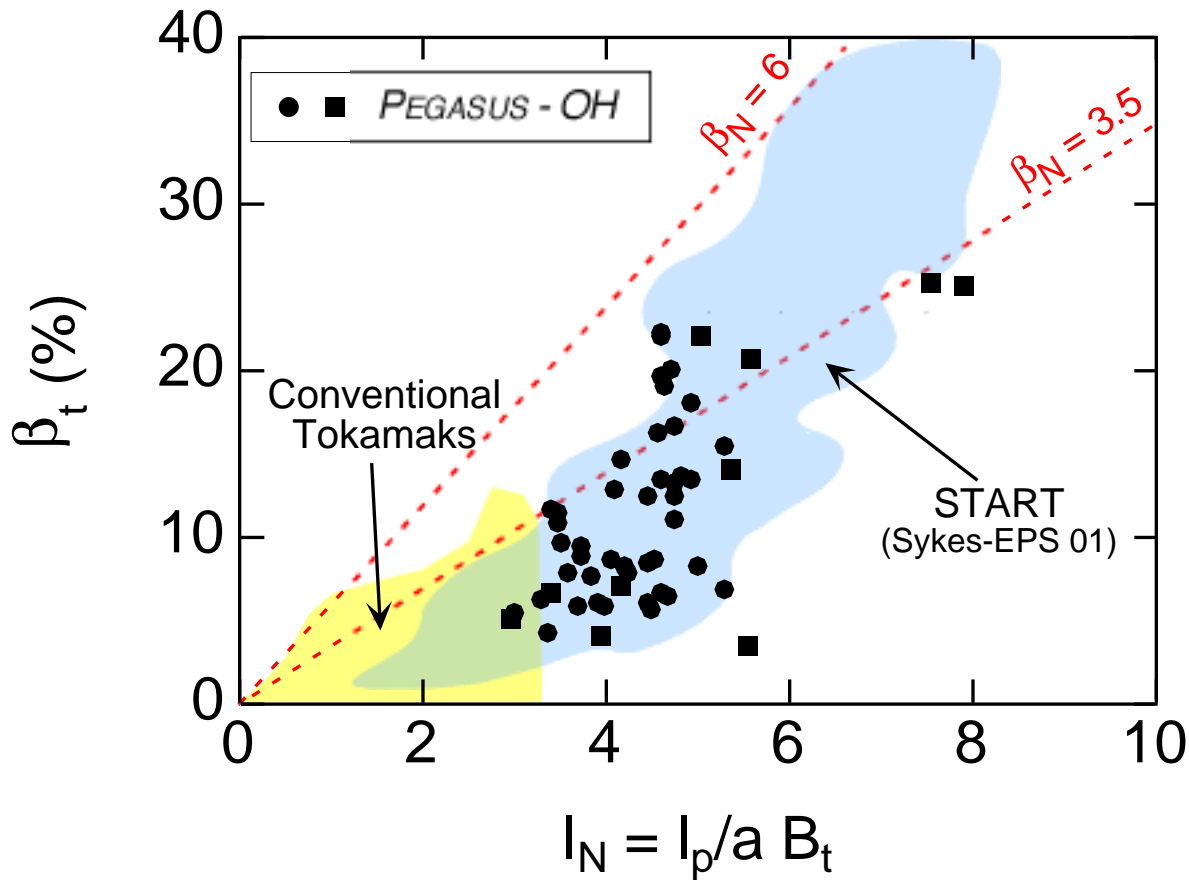


Constraints:
Rogowski Coil
18 Flux Loops
3 B_p Coils
Diamagnetic Loop

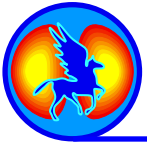


Pegasus is Accessing High- β_t ST Regime

- High β_t attained at high density, low-TF
 - Ohmic heating only; constant TF

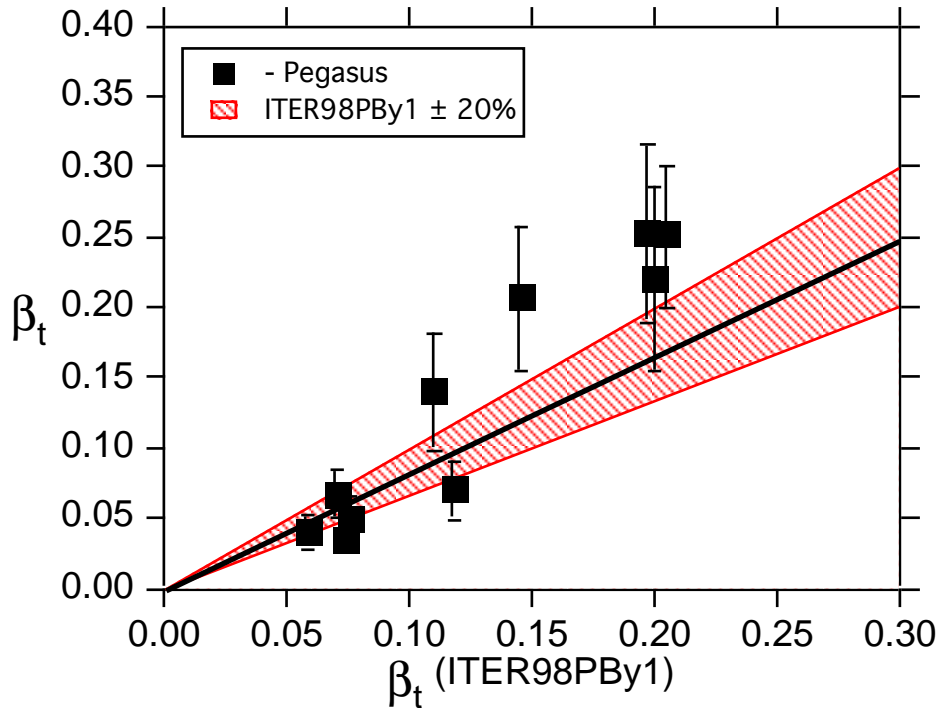


- Tools to move further under development
 - Additional MHD diagnostics
 - Increased V-sec
 - Increased, t -variable TF
 - HHFW heating
 - Density control

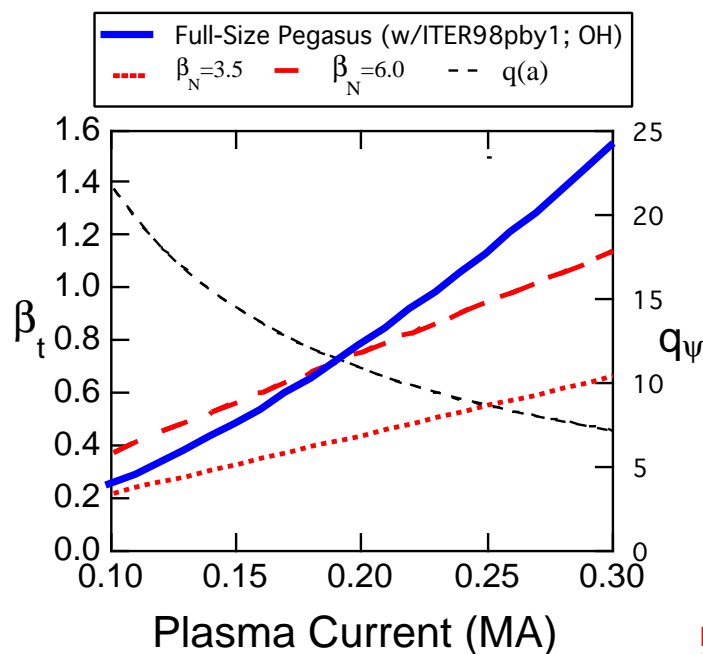


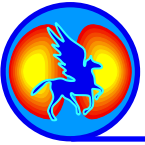
Estimates of β_t Consistent with START Confinement

- β_t estimated from magnetic equilibrium reconstruction



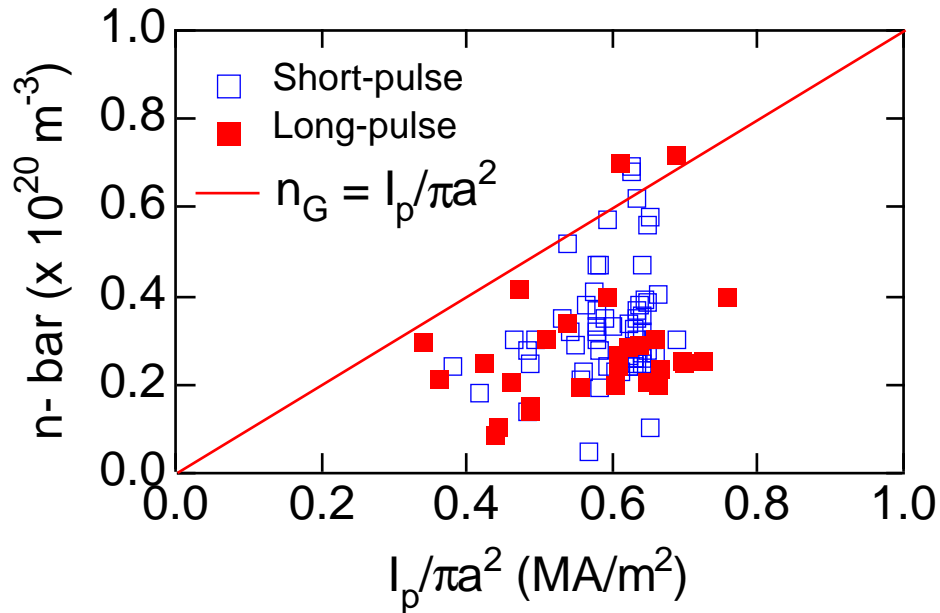
- High β_t accessible at full OH power; higher T_e with aux heating





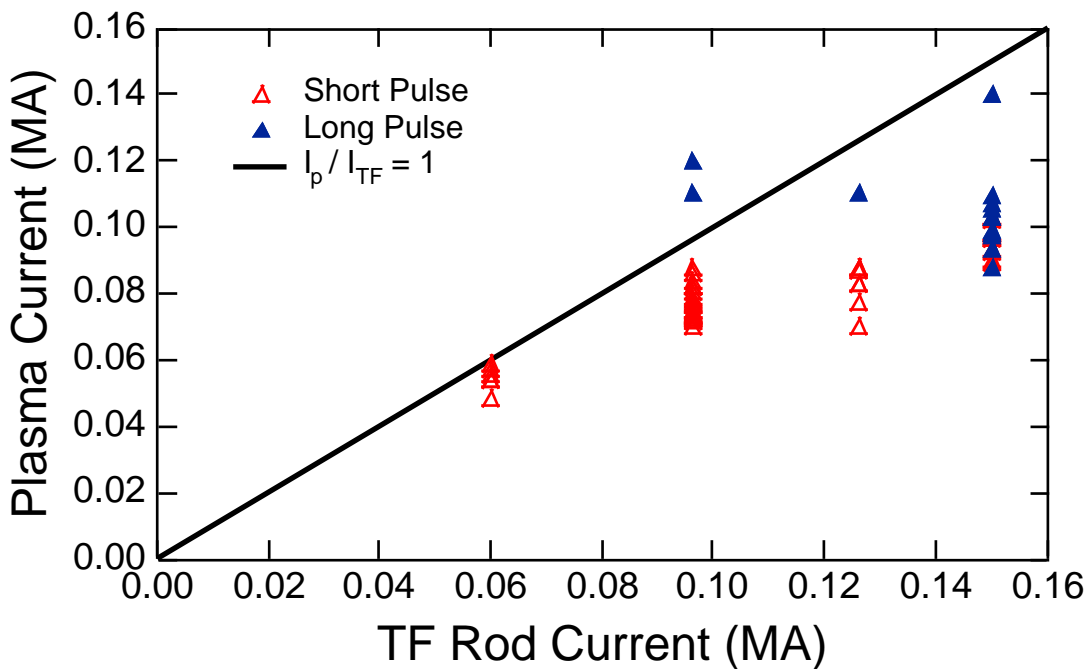
High Density, Low-TF Operation

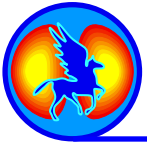
- Density Approaches the Greenwald Limit



- $I_p/I_{TF} \rightarrow 1$ at lower TF settings

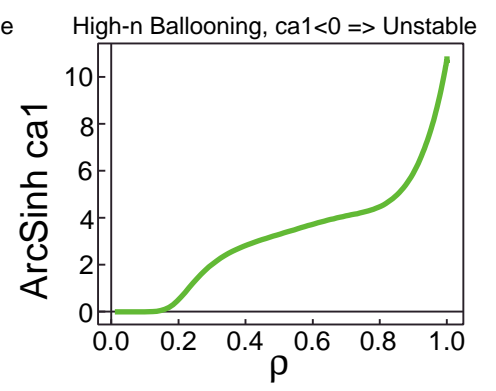
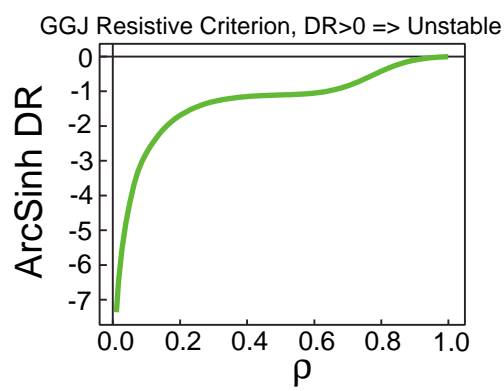
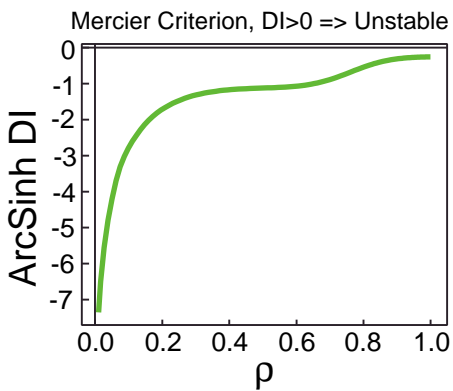
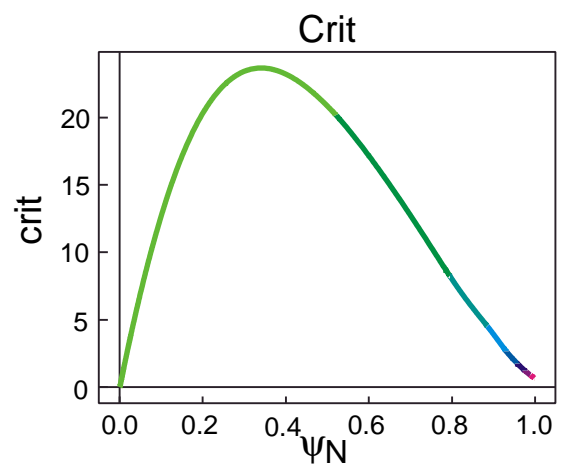
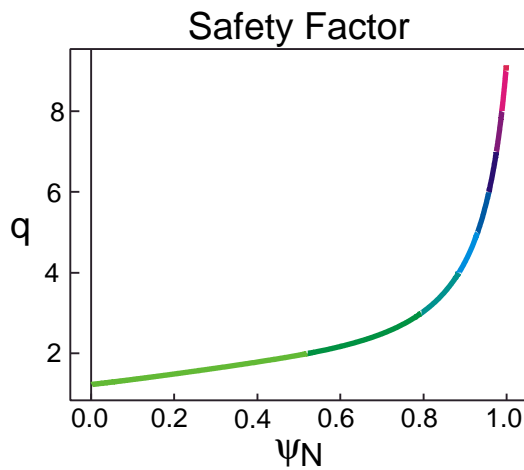
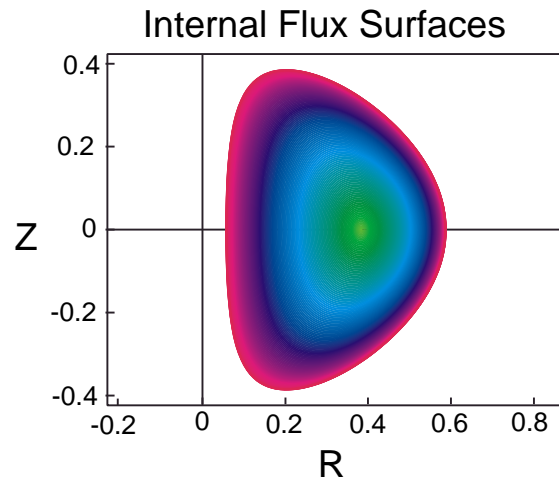
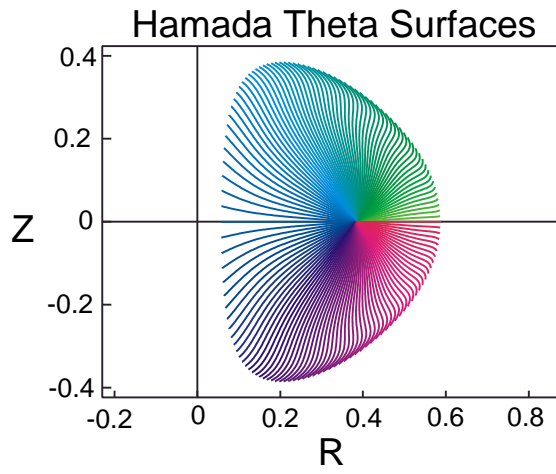
- MHD modes limit I_p ramp rate \rightarrow high I_p/I_{TF} only at low TF
- Present limits on I_p/I_{TF} : ramp rate, MHD, V-sec

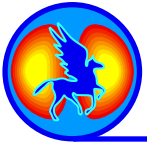




DCON is being implemented to map stability space and analyze individual equilibria

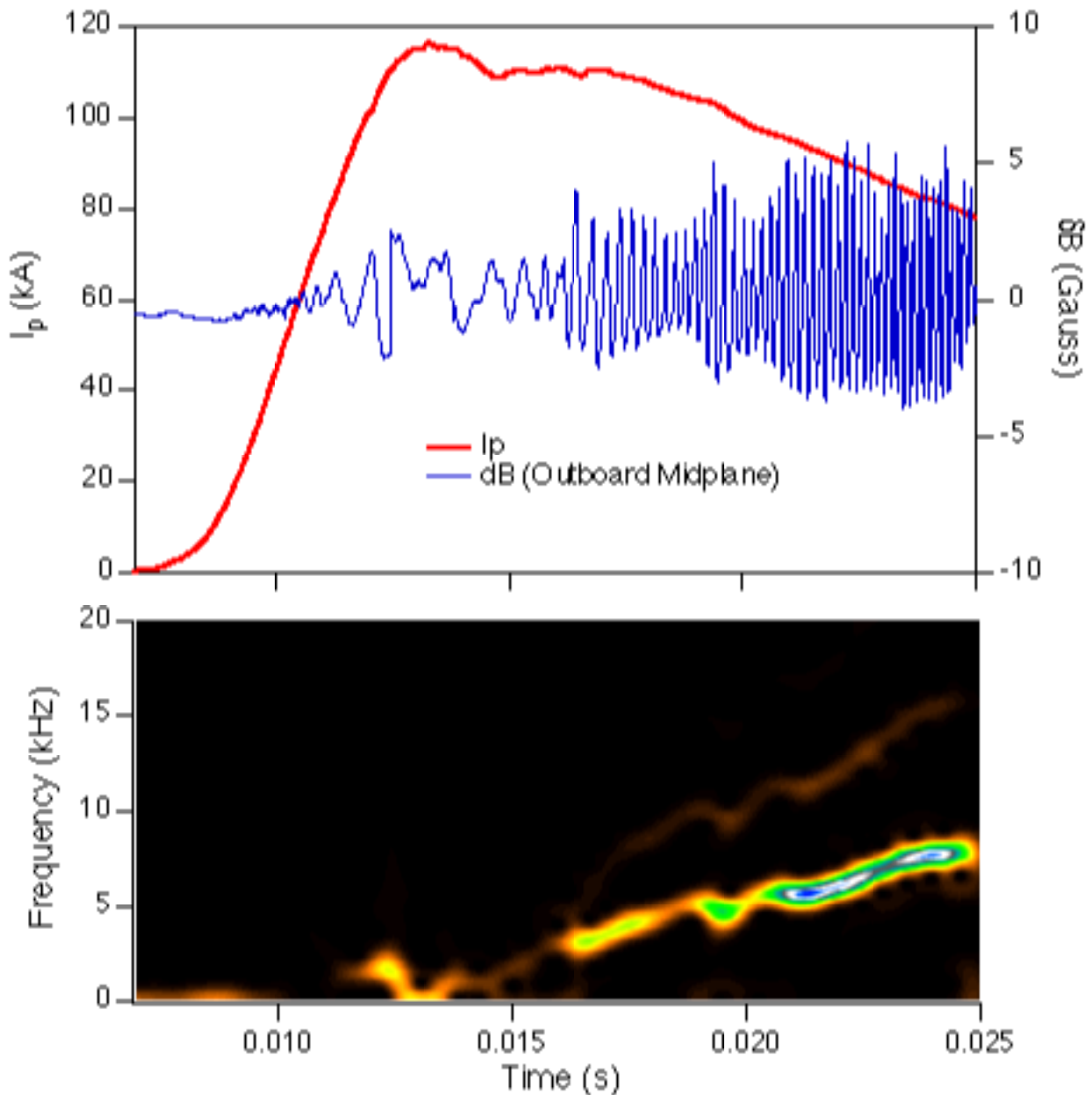
• Shot 12445, 18% β_t

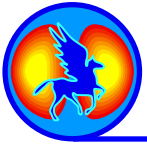




Significant MHD is observed during discharge evolution

- A rotating 2/1 mode is present in most discharges studied
 - Mode rotates in electron diamagnetic direction
 - Frequency is 5-10 kHz
- A lower frequency mode is often observed during the current ramp
- IREs and double tearing modes also observed

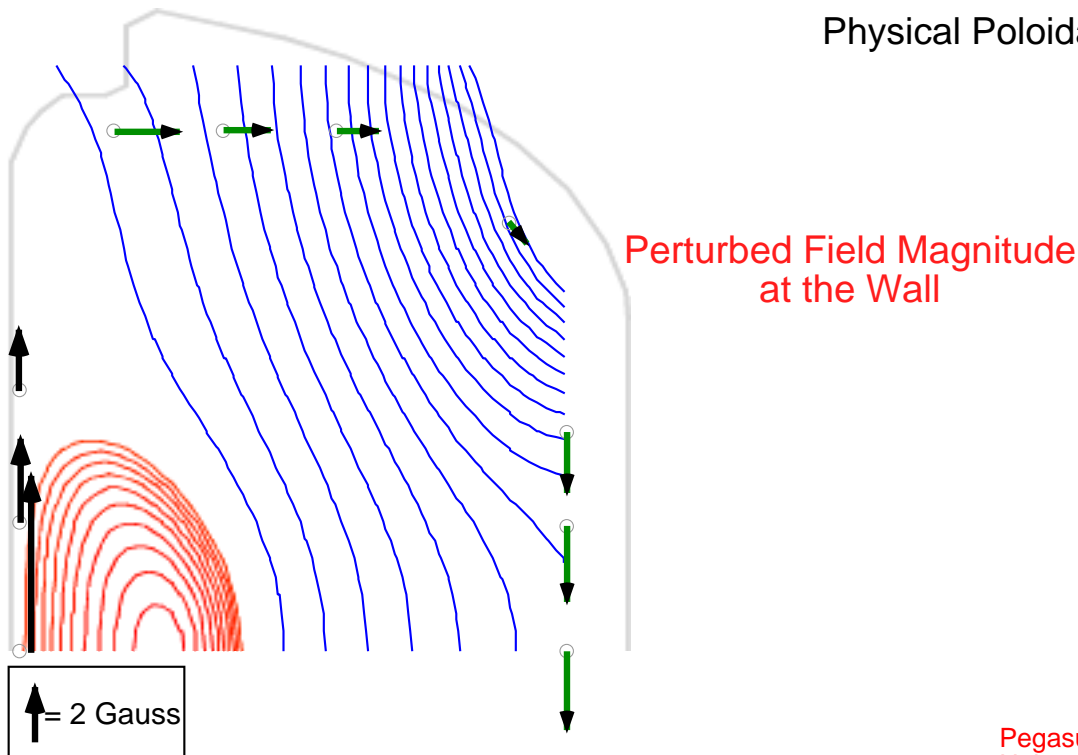
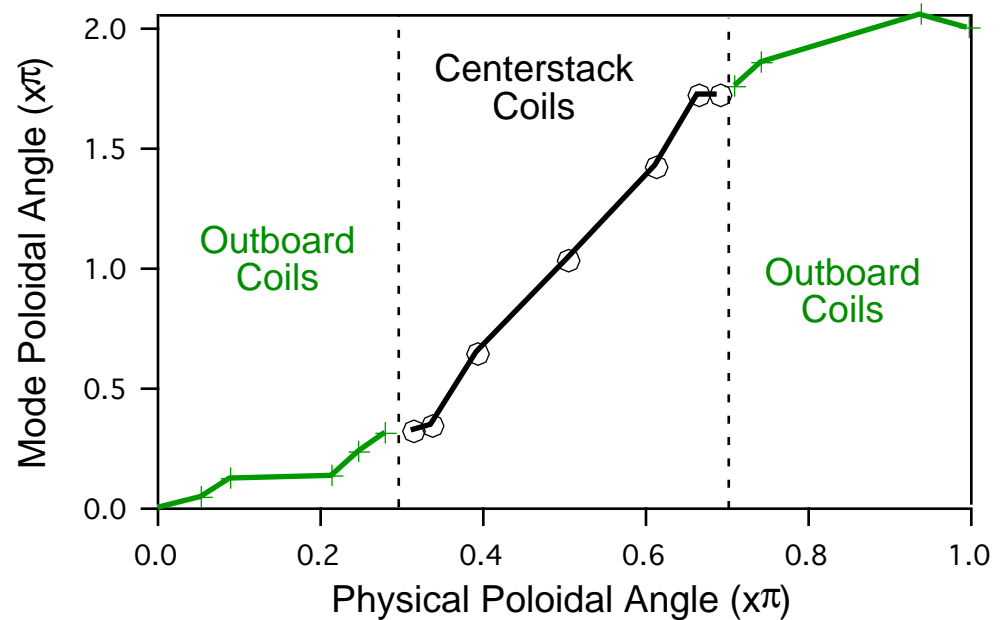


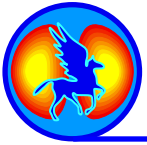


2/1 mode structure is poloidally asymmetric

- Poloidal and toroidal phase analyses clearly indicate mode is $m=2/n=1$
- Toroidicity of mode is seen in large phase shifts along centerstack
 - Roughly 1.5 wavelengths observed across 120° poloidally
- Mode is strongest on the low-field side
 - As plasma grows, LFS signal increases and HFS signal decreases

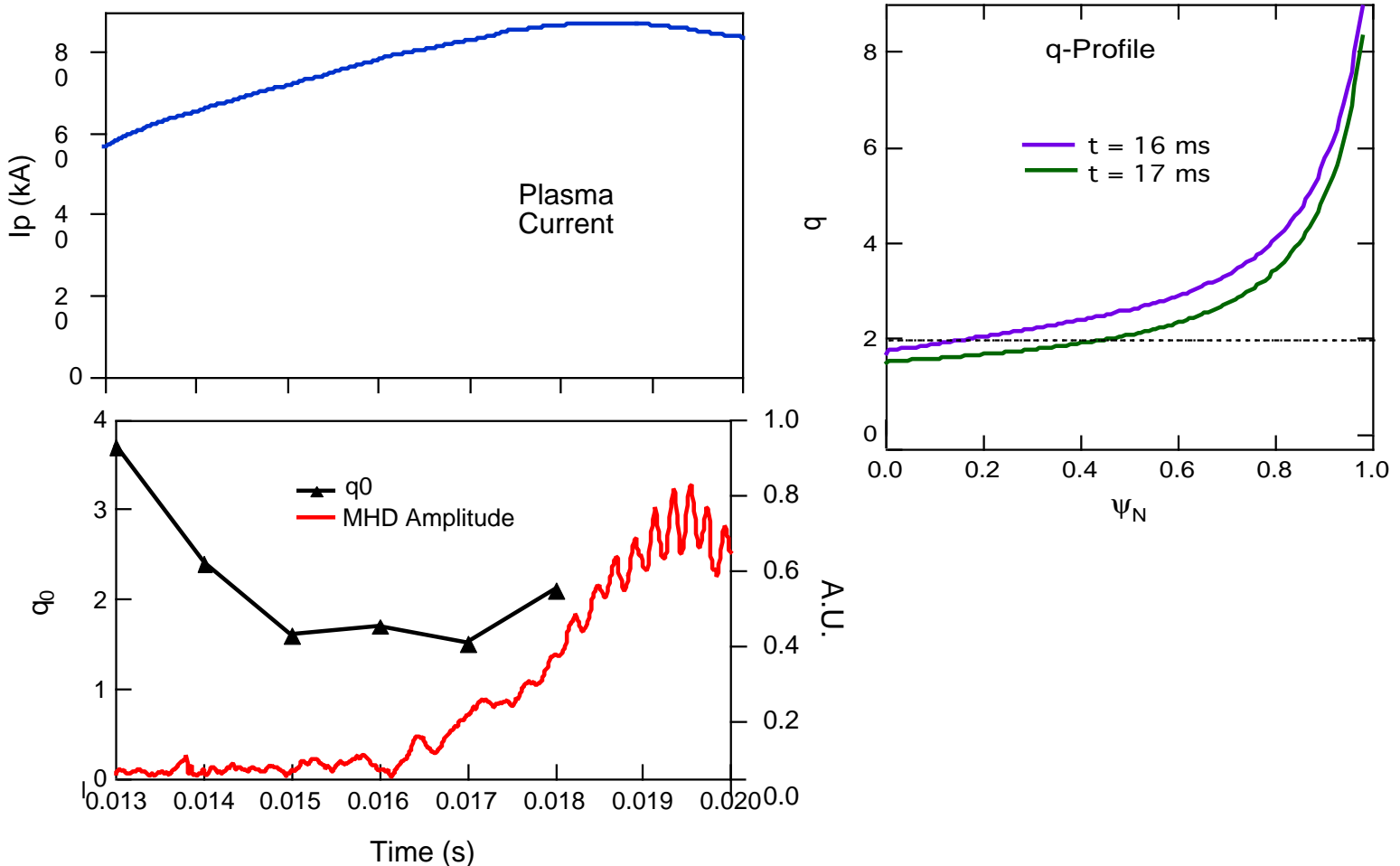
2/1 Poloidal Phase at the Wall



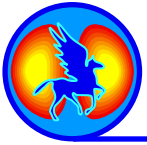


2/1 Mode appears and grows after $q_0 = 2$

- Consistent with observed helicity of the mode
- Central q presently inferred from Equilibrium fit
 - 2D SXR camera will constrain q_0



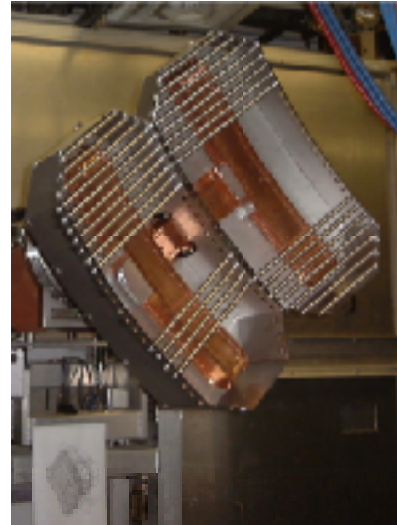
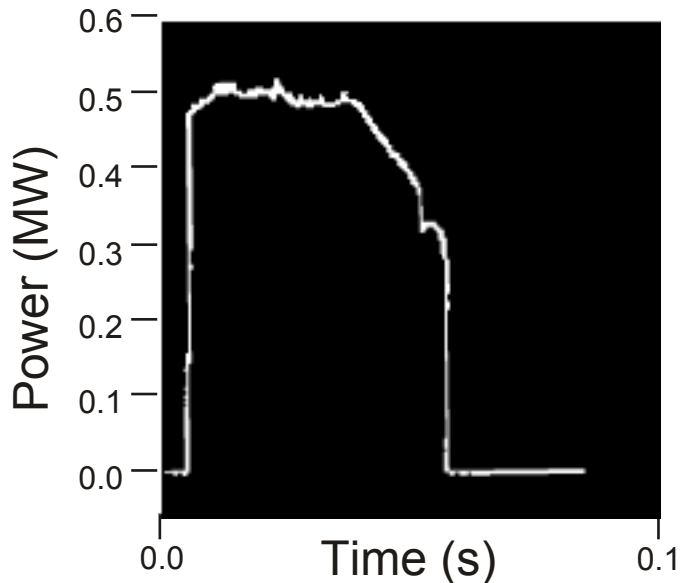
- In general, strong MHD activity appears to relate to I_p limits
 - Relative roles of MHD and V-sec limits under study



RF Startup and CD Techniques for High β Access

- **HHFW system installed and heating tests underway**

- $P_{RF} = 1\text{-}2\text{ MW}$ available; sufficient to access high β_t regime



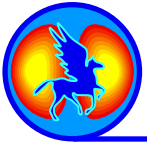
RF forward power results from
~ 50 ms test into dummy load

- Initial loading tests give an impedance of about 1 Ohm
- Up to 80 kW injected into vessel

- **HHFW Startup and CD applications:**

- Startup assist via preheating and/or current profile “freezing”
 - *Startup plasma phase: 40% single pass absorption*
 - *High β plasma phase: 100% single pass absorption*
- CD possible with present power supply and new antenna

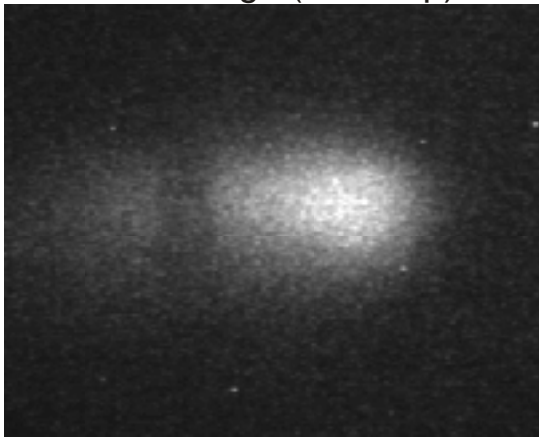
- **EBW heating and CD to be tested for application to high β , A \rightarrow 1 plasmas**



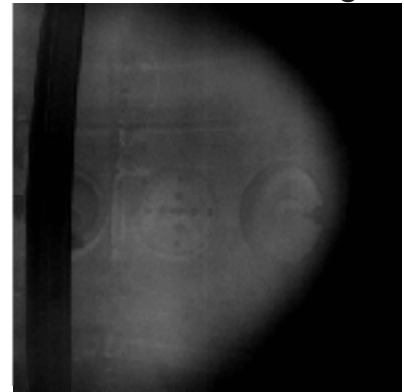
SXR Camera Provides $q(0)$ Constraint in Equilibrium Reconstruction

- **Tangentially viewing Soft X-ray camera provides measure of internal plasma flux surface shape**
 - Internal plasma shape strongly dependent on current distribution
 - SXR image added to measurements fitted in magnetic reconstruction
 - Initial data may give constraint on allowed $q(0)$

SXR Image (5ms exp)

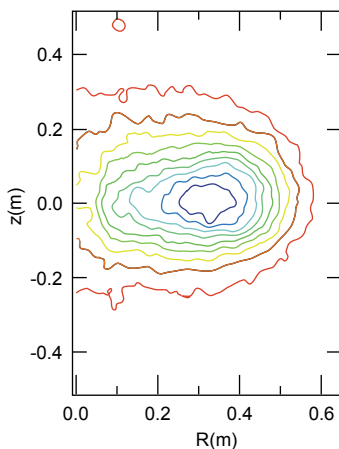


Visible Camera Image

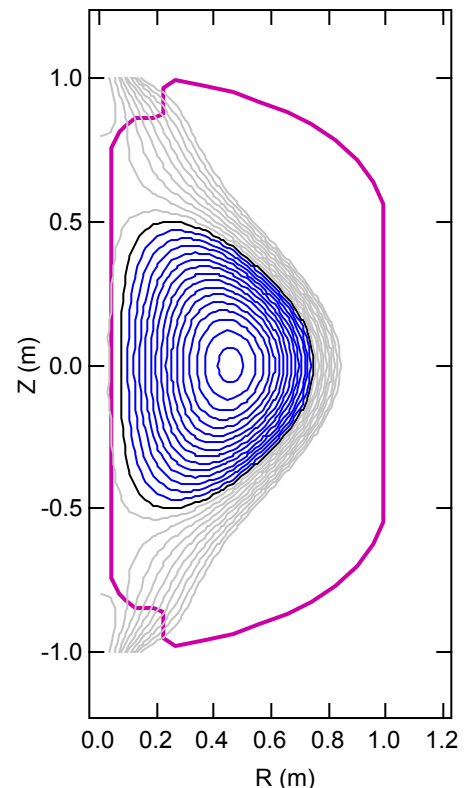
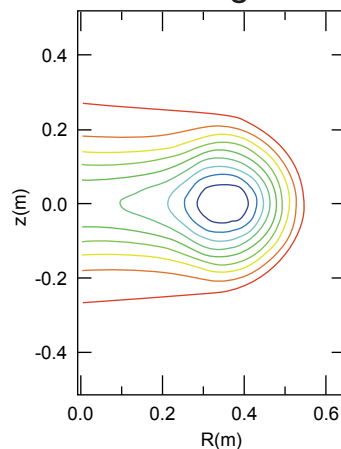


Shot 9639 (t = 18 ms), Frame 13

SXR Image Contour Plot



Calculated SXR Image



- Center-weighted fit gives $q(0) = 1.8$
- *Analysis development in progress*



Summary

- **Primary goal is to explore the $A \rightarrow 1$ regime**
 - Tokamak/Spheromak Overlap: How close can $A \rightarrow 1$ and maintain good stability and confinement?
 - Geometry (A , κ , separatrix) and current profile (ℓ_i , q_0 , q_ψ) influence on the stability limits?
 - Tradeoffs between $A \rightarrow 1$ and current drive requirements?
- **Ohmic campaigns have demonstrated low- A phenomena**
 - $I_p/I_{TF} > 1$, $I_N \approx 8$
 - $\beta_t \approx 25\%$, $\beta_N \approx 5$
 - $N_e \approx N_{GW}$
 - Formation at very low B_t (~ 0.04 T)
 - MHD activity: 2/1 mode, double tearing, IRE
 - Presently limited by MHD activity and available V-sec
- **New capabilities are rapidly coming online**
 - New magnetics diagnostics and PFCs installed
 - HHFW antenna testing during plasma ops
 - Upgrades for OH and TF in Fall 2001
 - Increased I_p , T_e , $q(0,t)$ to control MHD*
 - Several new diagnostics in development



PEGASUS Staff

- Merger of Research & Education Missions of UW

- **Pegasus Personnel - Experiment Team:**

Staff: (full)

G. Garstka
B. Lewicki
B. Ford
G. Winz

(part)

R. Fonck
P. Nonn (+ MST)
P. Probert (+ HSX)

Graduates:

R. Schooff
C. Ostrander
E. Unterberg
A. Sontag
K. Tritz

Undergraduates:

S. Diem	D. Schuster
M. Reinke	R. Lawson
J. Boerner	S. Schwab
T. Reinecke	






- **Associated Theory (CPTC)**

J. Callen	K. Comer
C. Hegna	



PEGASUS has Benefited Greatly from Contributions from Members of the Fusion Science Community:

• Collaborations

	NHMFL:	Solenoid design, fab., tests
		Stress Analyses; VV construction;
	PPPL:	Theory (MHD, RF...); Future expts.
	UCLA:	RF; Power Engr.; DNB assistance
	MST:	μ wave interferometer
		Engineering; diagnostics; e-gun j sources

• Contributions

NHMFL	Magnets
General Atomics	Vacuum Vessel, Iron core
PPPL	Capacitors; diagnostics; CAMAC
LANL	Caps; Ignitrons, RF systems
MST	Ross diodes; iron core; caps, etc.
HSX	EF cap bank
LLNL	Caps; DNB power system
ORNL	Thomson scattering
Westinghouse	High-E cap bank
UW SC Lab:	TF hex conductor



Plasma Diagnostic Set

• Presently operating diagnostics

<u>Diagnostic</u>	<u>Measures</u>
Core Flux Loops	V_L
Ext Flux loops	Vessel currents
Int. Flux loops	Equilibrium
Rogowski Coils	I_p
Diamagnetic Loop	Toroidal Flux / β_p
B_p , Mirnov Coils	Equilibrium / MHD activity
VUV (SPRED)	Impurity monitor
Filterscopes	Oxygen, Carbon, D_α
Interferometer	N_{e^l}
High Res. Camera	Plasma shape/position

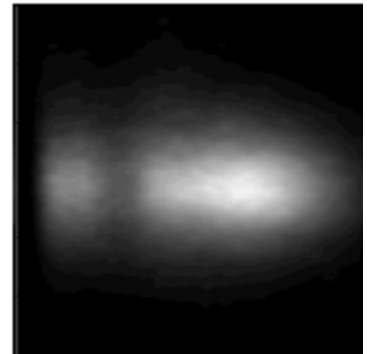
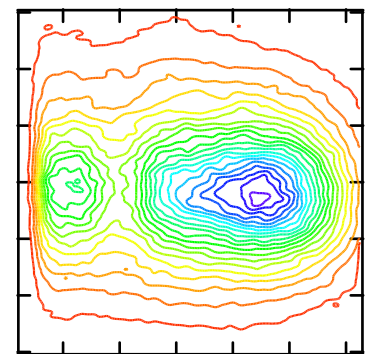


Image from 2D SXR Camera



Constant emissivity surfaces

• Near-future diagnostics

<u>Diagnostic</u>	<u>Measures</u>	<u>When?</u>
Tang. SXR Camera	Plasma shape/equilibrium	Summer 2001
SXR Array	Internal MHD	Summer 2001
Bolometer Array	P_{rad}	Fall 2001
VB Array	$N_e(R,t)$	Fall 2001
Ross Filters	T_{e0}	Summer 2001
2-Color X-ray	T_e	Fall 2001
Wall Pickup Coils	Vessel currents	Winter 2001
DNB	$N_e(R,t), T_e(R,t), j(R)$?
EBW Radiometer	$T_e(t)$?